

Current developments

At the Medical University of Vienna

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Overview

- Neutron measurements and simulation
- Cross section evaluation
- Implementation of microdosimetric calculation function

Neutrons production during ion beam therapy

- Neutron production during ion beam therapy
 - Increasing interest of community
 - Pregnant patients, etc.
- Experimental measurements
 - Water phantom @ isocenter
 - Central beam of P, C12, He ions
- Cross-comparison MC tools
 - Fluka
 - Geant4/OpenGATE



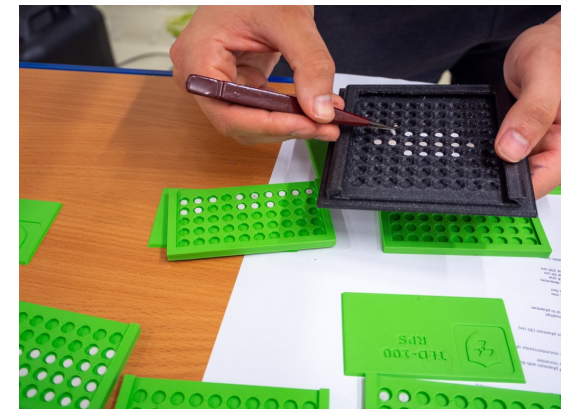
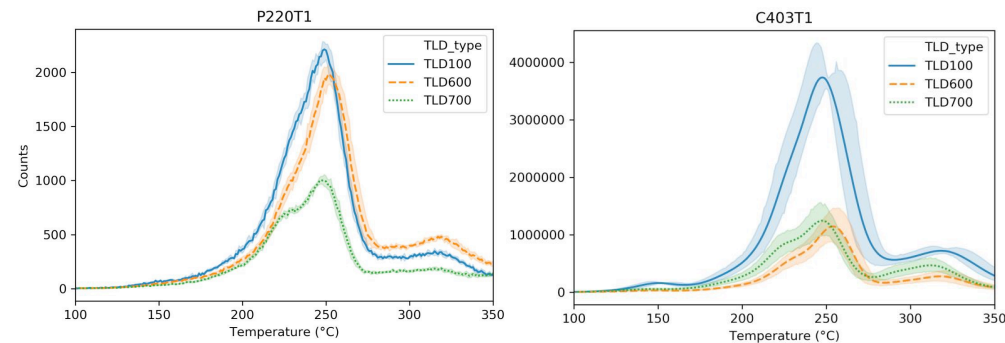
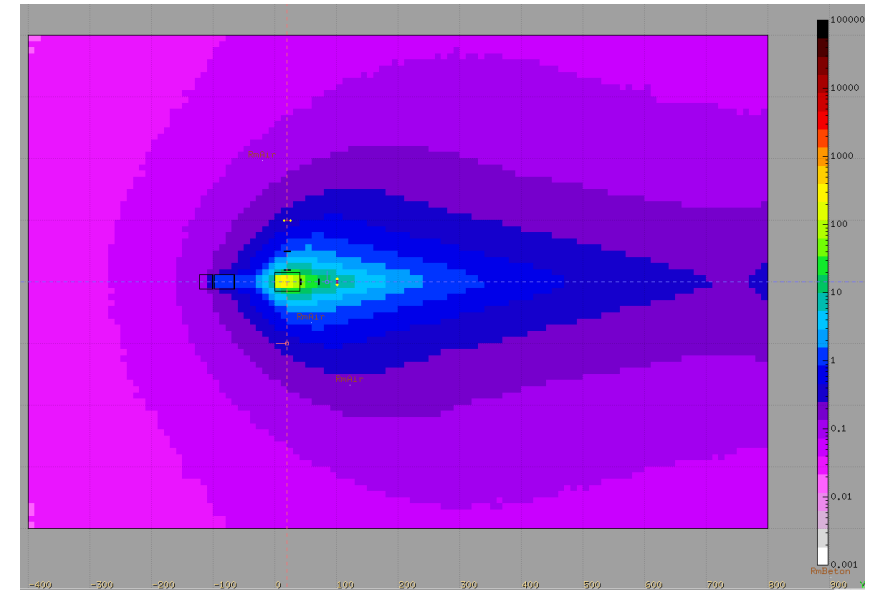
Experimental Set- Up

- **Multiple particles and energies**
 - P 142.5, 220 MeV
 - He 140, 220 MeV/u
 - C12 273, 402 MeV/u
- **Detectors**
 - WENDI 2 (Thermo Fisher)
 - TLD 100 + 600/700
 - calibrated for thermal neutrons
 - Nuclear reactor @ Vienna
- **Positions**
 - Lateral (15, 50, 100 cm)
 - Tail (40, 70, 120 cm)



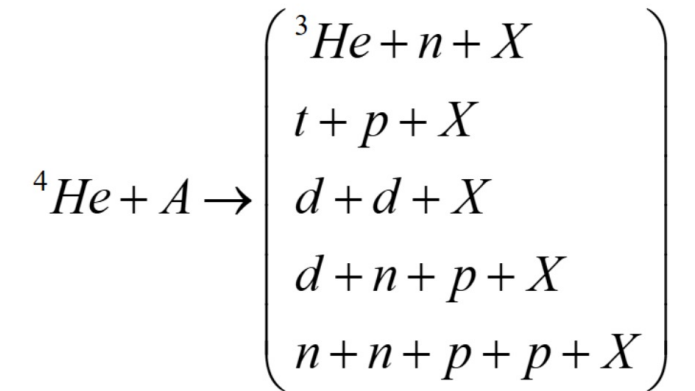
Preliminary comparison

- Similar results for experimental measurements and simulations
 - For WENDI 2
 - Lateral only
- Issues:
 - TLD in tail unreliable
 - Not only neutrons...



Cross sections

- Helium ions available at MedAustron
- Cross Section Data limited
 - Helium ions in clinical energy range
- Evaluation of influences on clinical parameters
 - Dose distribution
 - Particle flux
 - Neutron flux
 - LETd
- GOAL: Modify individual Cross Sections for Sensitivity Analysis
 - Mainly interested in projectile fragments of ^4He



Physics Wrapper

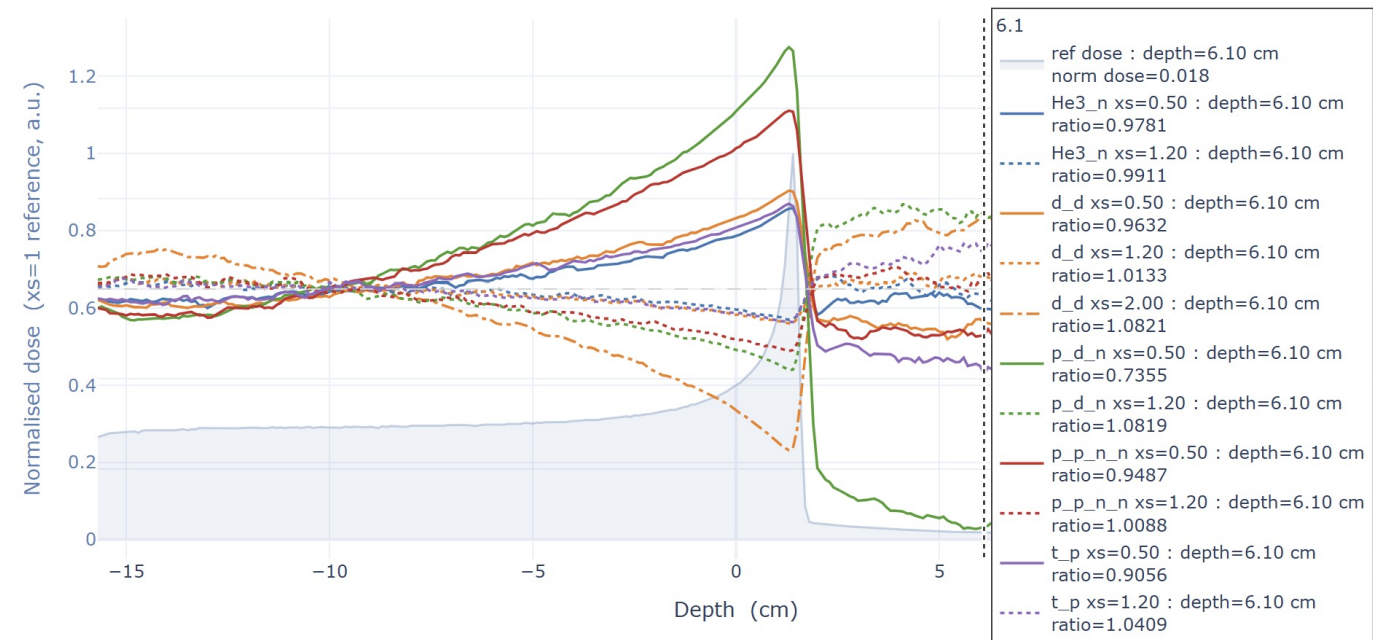
- Developed a physics wrapper
 - Scale cross sections
- Choose a channel (e.g. $4\text{He} + X \rightarrow t + p + X$) to scale by choosing:
 - Nuclear process to scale \rightarrow alphaInelastic
 - Channel signature in the format $[[Z,A],[Z,A],\dots] \rightarrow [[1,3],[1,1]]$
 - Cross Section Scaling factor can be either <1 or >1
 - Optionally: Set channel definition to exclusive (default is inclusive)
 - Define energy range of specific secondaries $[E_{\text{min}}, E_{\text{max}}]$ to be scaled \rightarrow useful to only scale projectile or target fragments

Physics Wrapper II

- Replaced methods:

- PostStepGetPhysicalInteractionLength → Mean Free Path is scaled in order to increase process cross section if needed
- PostStepDolt → look at the proposed particle change → accept/rollback depending on conditions set

- Currently working on test cases



Microdosimetric calculations

- Calculation of microdosimetric spectra
 - Time consuming
 - Very small step/voxel sizes
- Hybrid approaches exist
 - Using MC to calculate particle flux
 - Using analytical formula from there
 - Sato et al, PMB 2023

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<https://doi.org/10.1088/1361-6560/ace14c>

Physics in Medicine & Biology



PAPER

Improvement of the hybrid approach between Monte Carlo simulation and analytical function for calculating microdosimetric probability densities in macroscopic matter

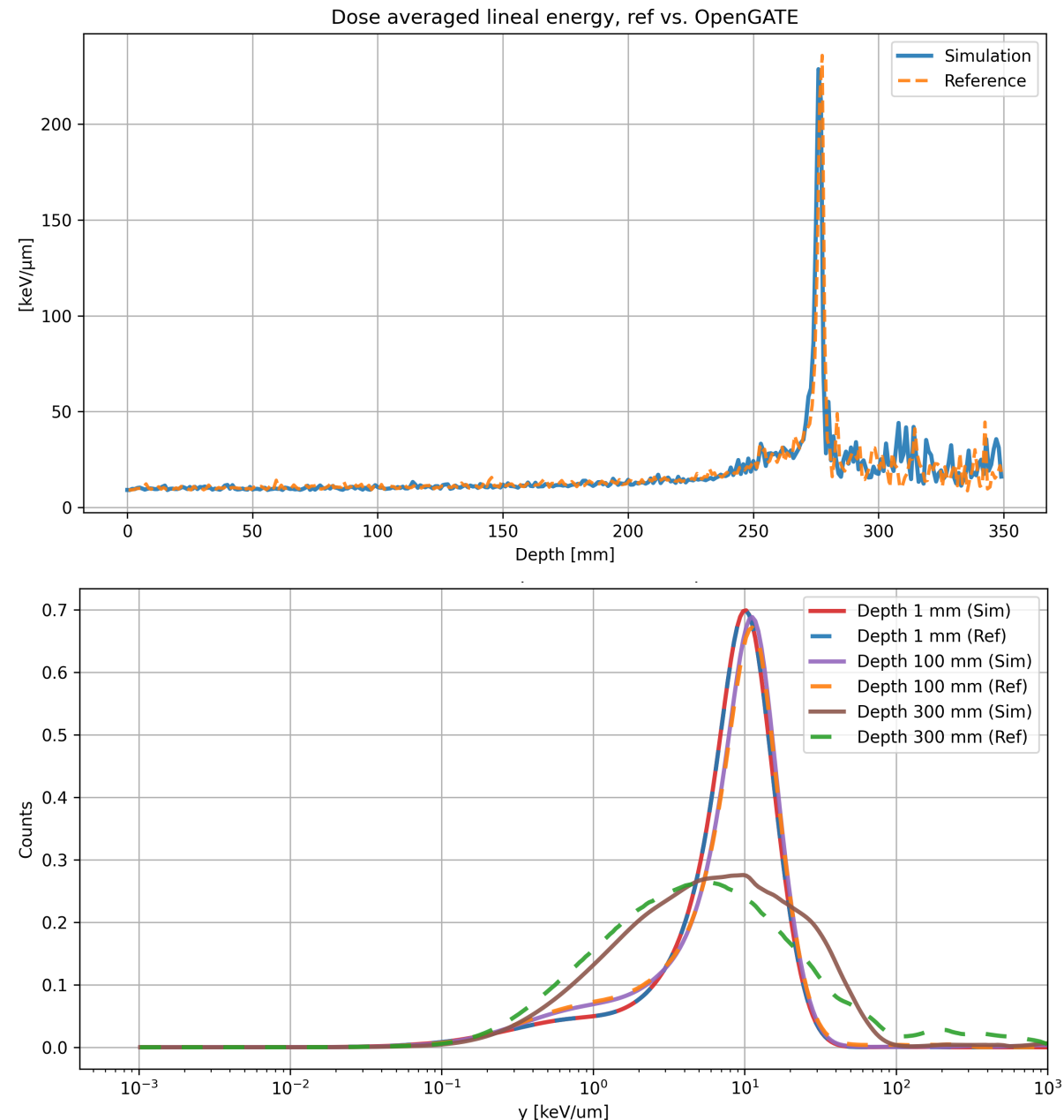
Tatsuhiko Sato^{1,2,*}, Yusuke Matsuya^{1,3}, Tatsuhiko Ogawa¹, Takeshi Kai¹, Yuho Hirata¹, Shuichi Tsuda¹ and Alessio Parisi⁴

Microdosimetric calculations

- Implementation into OpenGATE
- New dose actor
 - AMF – Abridged Microdosimetric Formula
- Good agreement with full simulation

- Outlook:
 - Perhaps to calculate biological models

- Working on test cases



Summary & Outlook



Summary

- **Neutrons**
 - Comparison simulation – experiments promising
 - Unsolved tail measurement problems
- **Cross sections**
 - Impact of cross section on clinical parameters
 - Ongoing
- **Microdosimetric functions**
 - Faster way to get spectra with MC
 - Input for biological modelling