



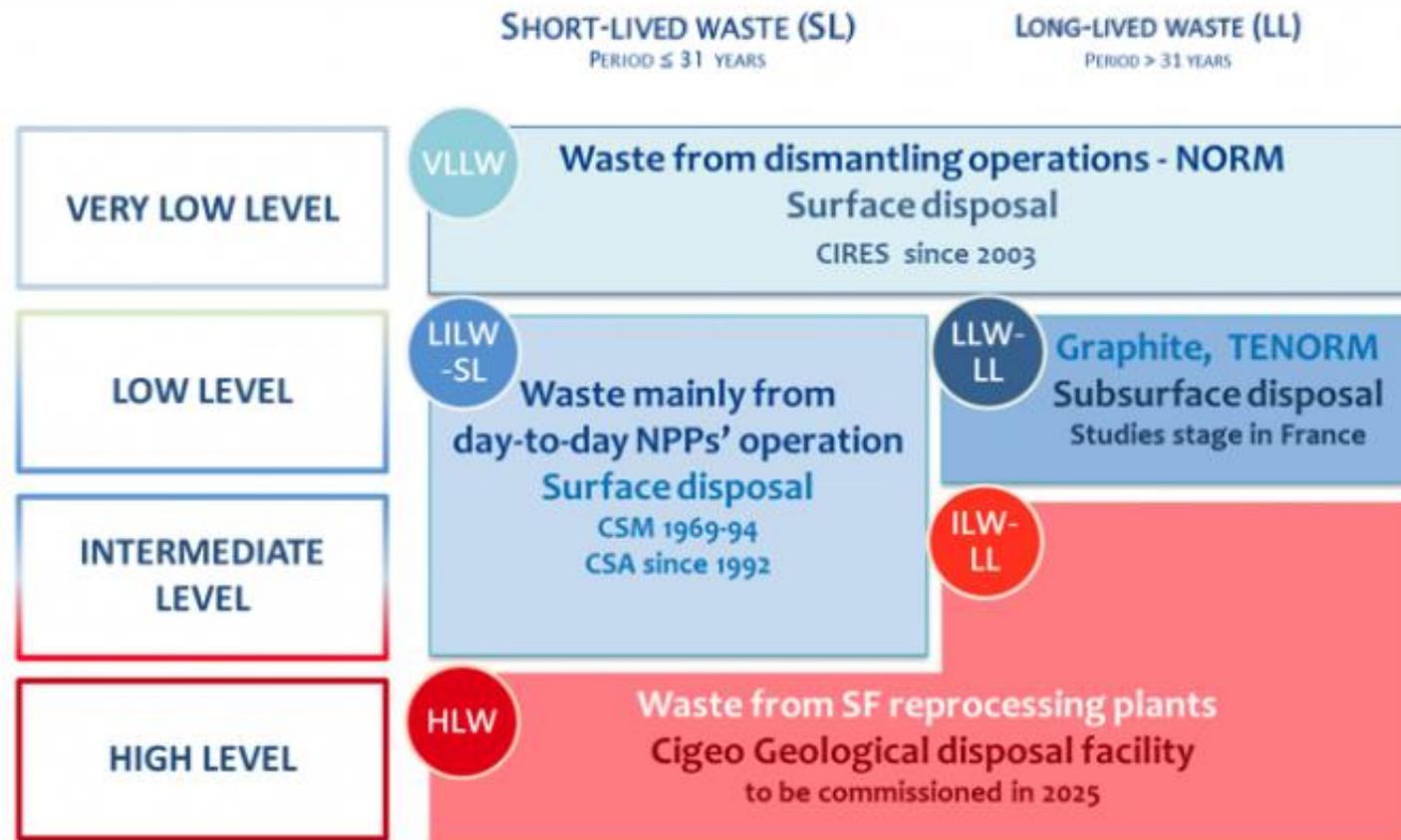
Internship defence

The Nuclear Measurement Laboratory

- ❑ 25-30 permanent staff
- ❑ 5-10 non-permanent staff (PhD, trainees, temporary research contracts)
- ❑ More than 50 projects or collaborations (on R&D and application topics)



Introduction



- Below 100-day period, management through in-situ radioactive decay.
- Only solid waste is to be disposed of.

Detection methods

Destructive: sampling the content of the package

- Inaccurate if heterogenous
- Radiological exposition of operators

Passive: detecting natural activity of package

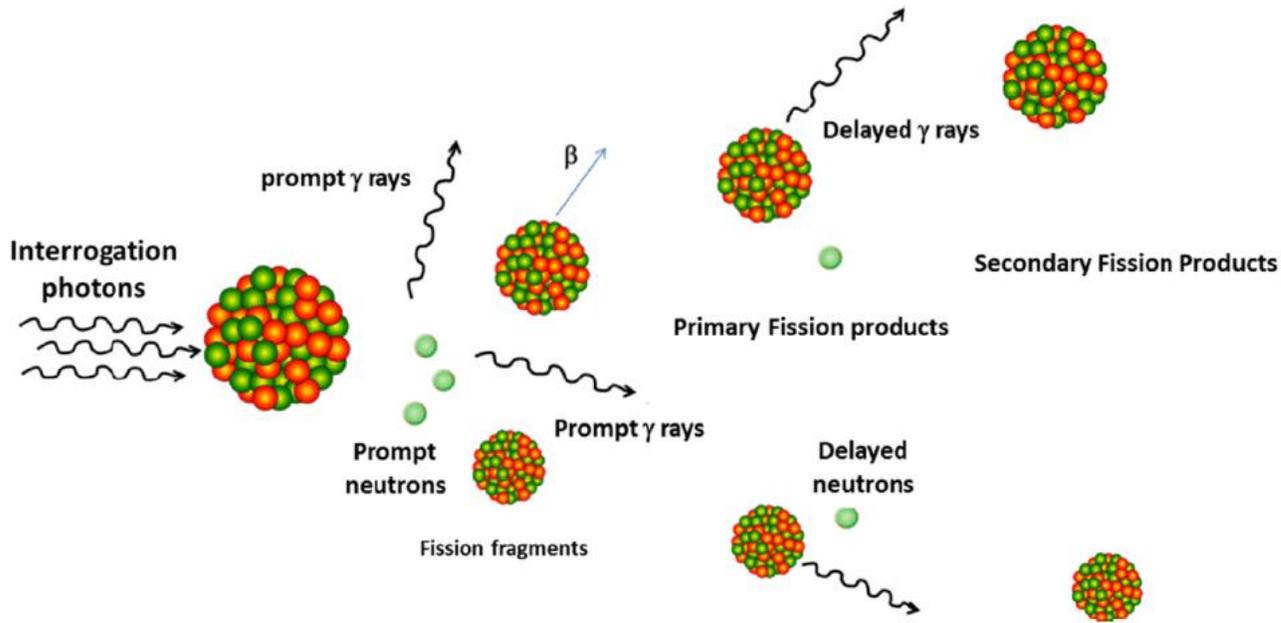
- Low energy
- Low activity

Active: probing the content with neutrons/photons

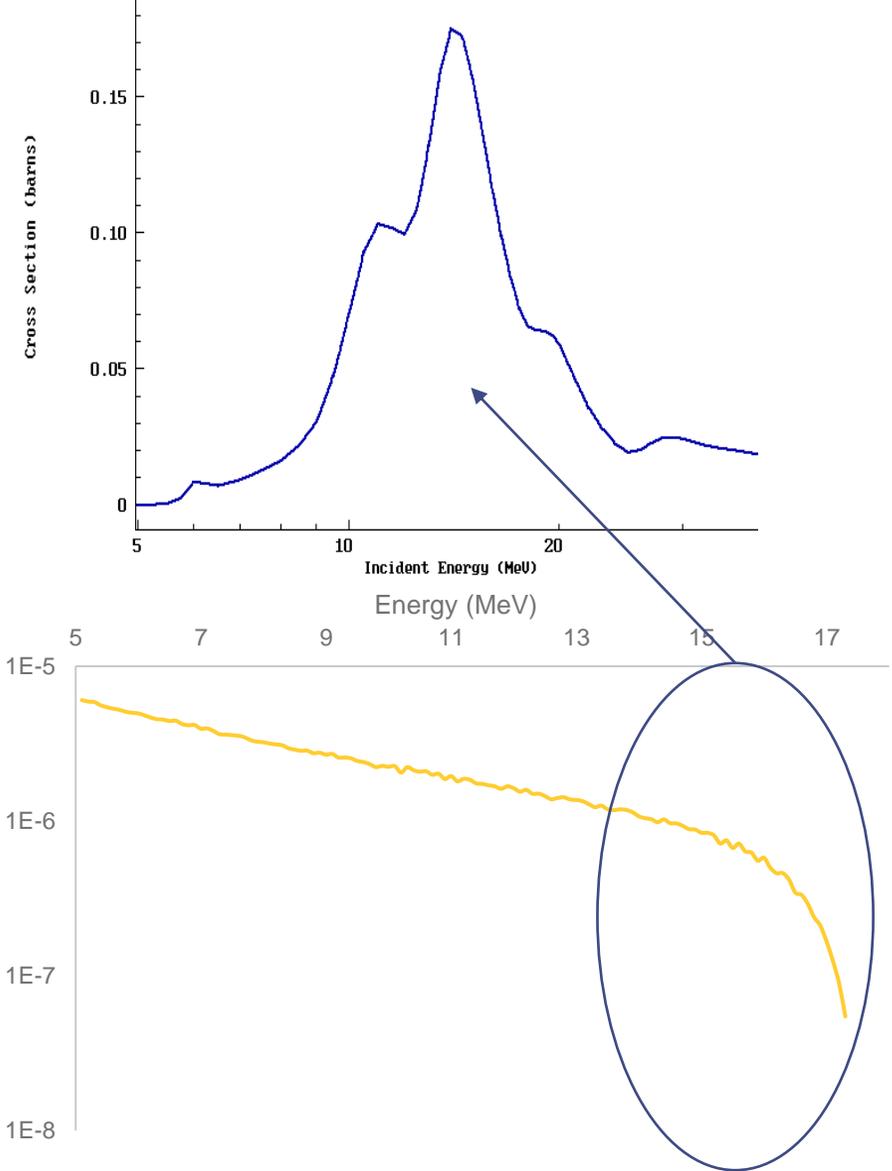


CEA 870L cemented waste drums

Physics behind API



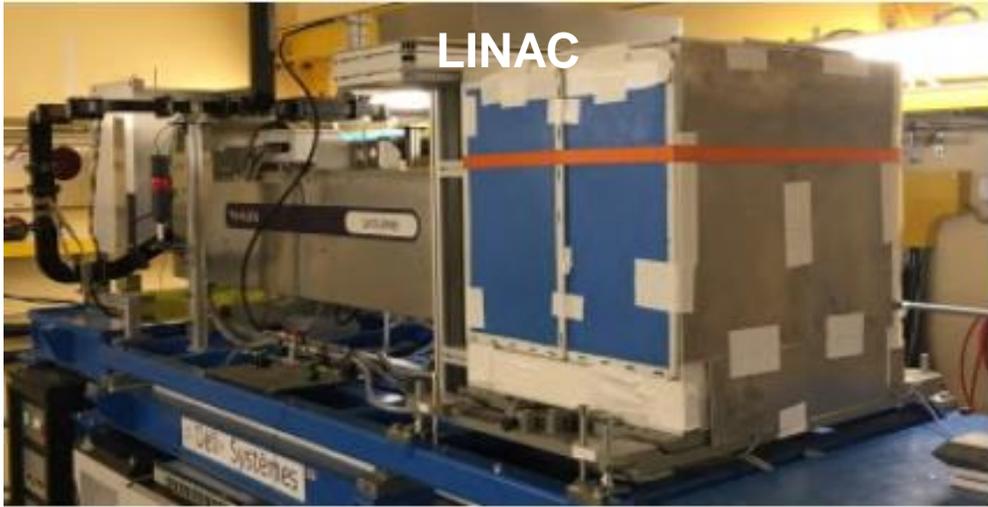
Photofission cross-section for U^{238}



LINAC x-ray spectrum

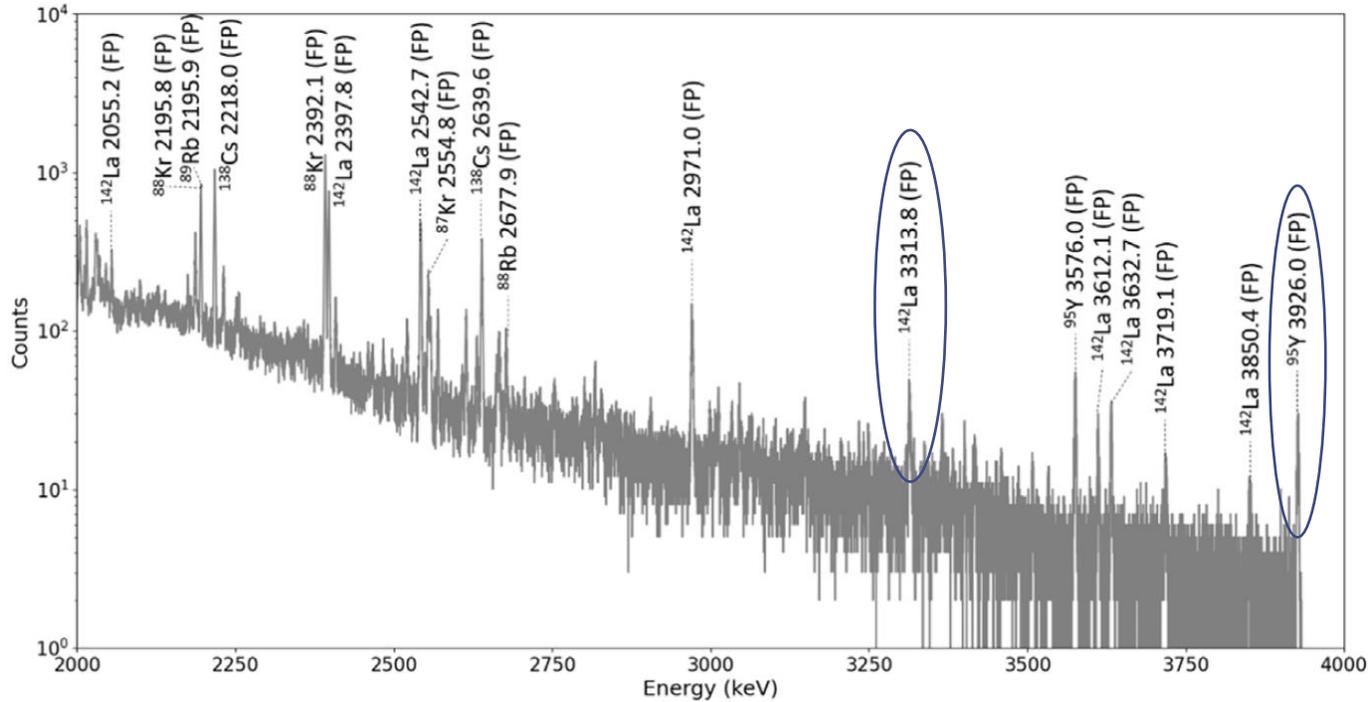
18/06/2024

HPGe photofission Setup for CINPHONIE casemate

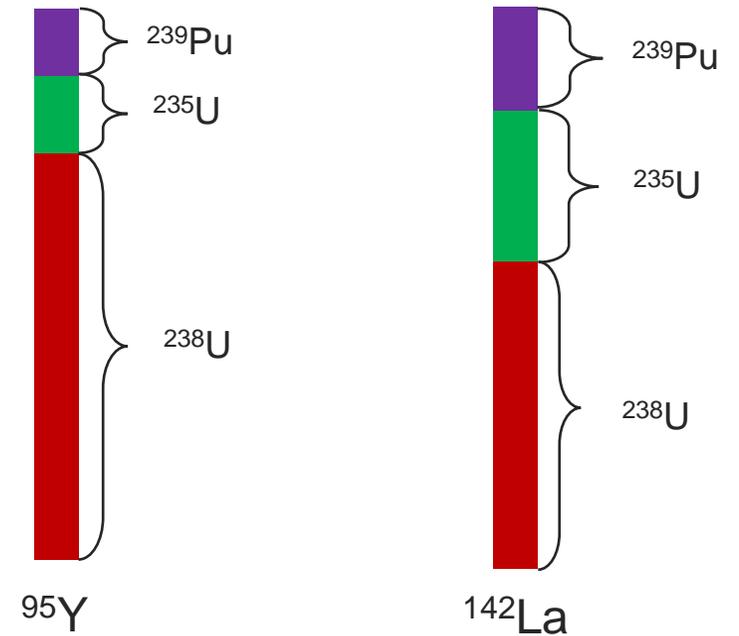


- Detect
- Localize
- Quantify

HPGe signal analysis



HPGe gamma spectrometry



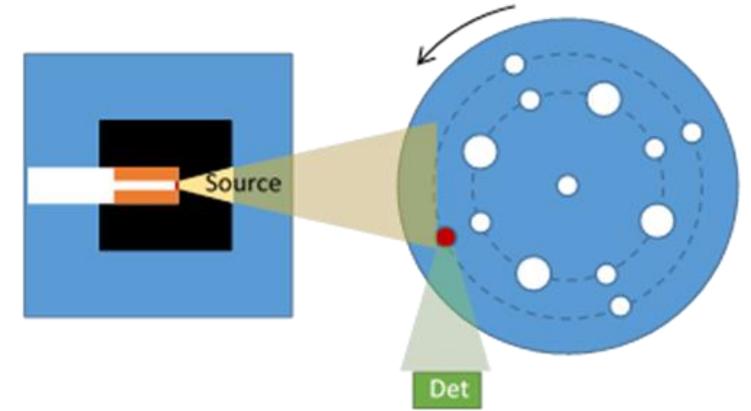
Quantity/Identify actinides

Hours of irradiation
 Days of measurement
 Detection limit: dozen of grams

Experimenting with LaBr_3 fast scintillators



- 1s irradiation
- 1s cooling
- 1s measurement



Nuclear Inst. and Methods in Physics Research, A 1033 (2023) 166860

Contents lists available at ScienceDirect
Nuclear Inst. and Methods in Physics Research, A
ELSEVIER journal homepage: www.elsevier.com/locate/nima

Full Length Article
Photofission delayed gamma-ray measurements in a large cemented radioactive waste drum during LINAC irradiation
C. Caracou^a, D. Eck, B. Geolot, E. Payan, B. Pérot, J. Roullot Rouaux, E. Simon
^aCEA, DSM, DSDM, Nuclear Technology Department, Cadarache, F-13108 St Paul Les Bains, France

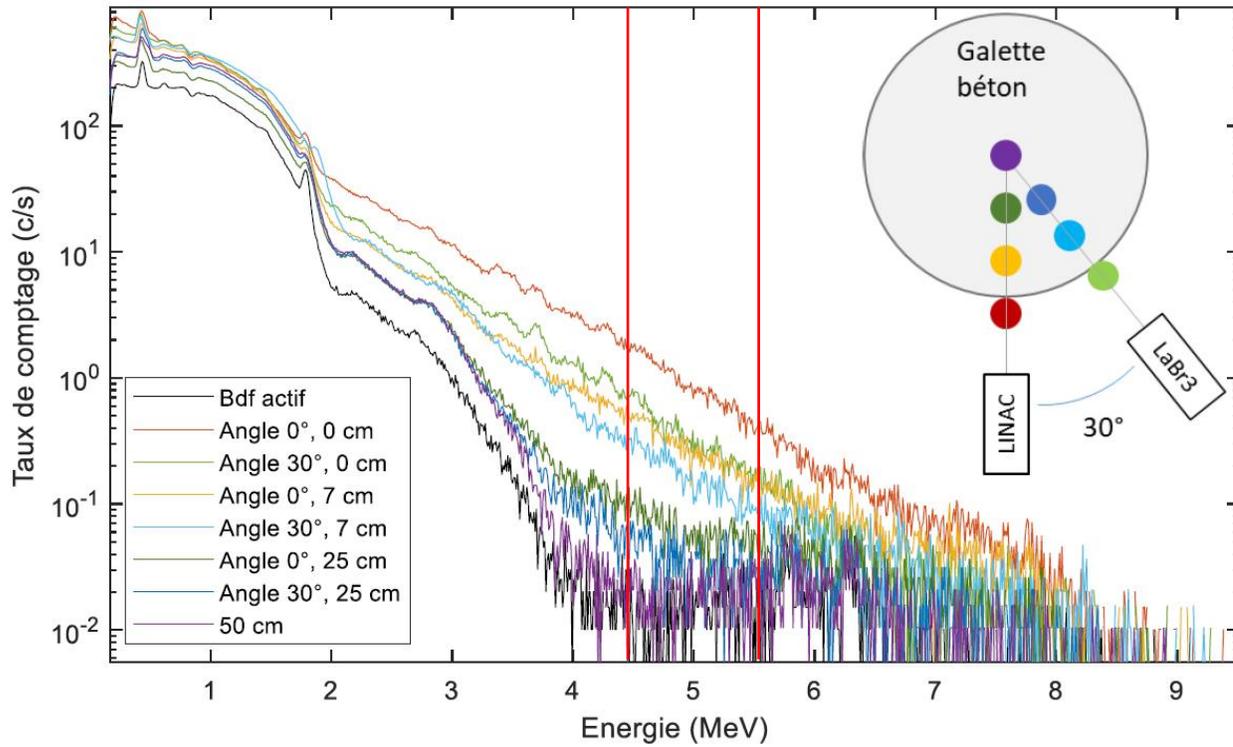
ARTICLE INFO ABSTRACT
Nuclear materials characterization in 870 L cemented waste drums is challenging due to significant neutron and gamma ray attenuation. Among the existing non-destructive approaches, photon active interrogation is of particular interest. At LINAC, neutron energies continue to reach greater than 10 MeV, leading to highly penetrating intergrading fissionable photons that can reach nuclear materials in the depth of the 870 L waste drum and allow the detection of delayed gamma rays emitted by photofission products. Because the intergrading photons and neutrons fluxes are extremely high during irradiation, measurements with high purity germanium detectors are usually performed after a long irradiation time, for example by moving the detector out of the waste drum. Therefore, such measurements are only sensitive to delayed gamma rays associated to photofission products having sufficient long half-life of at least years of seconds, which sets a valuable ceiling for fission products with a shorter half-life. In this paper, we report gamma rays measurements of very short lived photofission products using a 10 × 10 cm cylindrical $\text{LaBr}_3(\text{Ce})$ fast scintillation detector. A digital neutron sample was irradiated by a pulsed bremsstrahlung source with maximum energy 17.5 MeV in a mono pulsing mode (1 s irradiation, 2 s cooling). This allowed to acquire a strong delayed gamma signal within a time window of 2 s. A clear signature of nuclear materials was observed even in the depth of a concrete shielding, with an expected signal-to-noise ratio beyond 3.5. Moreover the background of non-nuclear activated materials is negligible. These results are compared to a semi-analytical model that qualitatively agrees but underestimates the measurements by a constant factor 1.2, probably caused by a wrong beam intensity normalization. Indeed, in this work the 2D/AC calorimetric structure (macro pulsing) was different from the nominal working point used to characterize the beam (total pulsing mode). Hence it allows direct calculation of the detector exposure to delayed gamma-ray photofission measurements, this model is a fair calculation alternative to time-consuming Monte Carlo simulations, in view of former studies that will allow this feasibility demonstration.

1. Introduction
Standard 100 L radioactive waste drums from the Commissariat à l'Énergie Atomique et aux Énergies Alternatives (CEA) having a dose greater than 2 Mrad/h at contact are compacted and embedded in a concrete matrix inside 870 L metallic containers [1]. The 870 L containers, referred as 870 L drums in the following, have an external diameter of about 100 cm and a height of about 115 cm. The total waste weight is comprised between 300 and 500 kg, approximately, with an apparent density between 1 and 2 g/cm³. The block is surrounded by a concrete envelope of about 5 cm in the periphery (see Fig. 1) and above the waste, and 10 cm below. The mass ratio of the 870 L drum after adding concrete is globally between 1.5 and 2 tons.
To obtain allowance for transporting a drum or for its acceptance in incineration storage or final repository, the waste producer must guarantee that the amount of fissionable material in the drum is below a certain limit, for instance 200 g [2]. In smaller radioactive waste packages, the amount of fissionable material is often measured by gamma ray spectroscopy and passive neutron measurements. However, given the large gamma and neutron attenuation in 870 L cemented drums, passive measurements lead to large detection limits, and even in the case of a detection, to unacceptable uncertainties related to the generally unknown position of nuclear materials. Active neutron measurements is also difficult due to the large attenuation of both intergrading and induced fission neutrons by the hydrogen made present in the concrete matrix. An alternative to neutron interrogation is to measure photofission delayed gamma rays induced by the high-energy bremsstrahlung photon beam of an electron Linear Accelerator (LINAC), both interrogating and induced photons having enough energy to reach nuclear materials in the depth and to escape the drum, respectively. The use

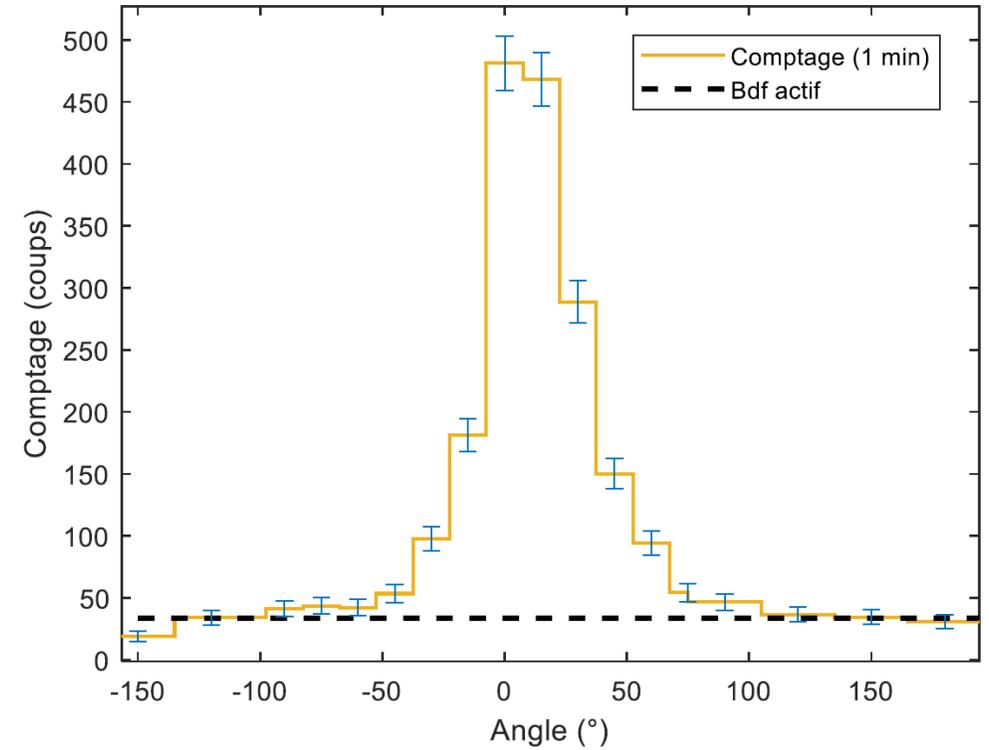
* Corresponding author.
E-mail address: cec@cea.fr (C. Caracou).

https://doi.org/10.1016/j.nima.2023.166860
Received 19 January 2023; Received in revised form 5 May 2023; Accepted 7 May 2023
Available online 12 May 2023
0168-9002/© 2023 Elsevier B.V. All rights reserved.

Results



10 min measurements

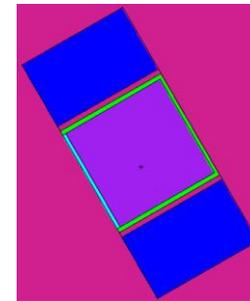
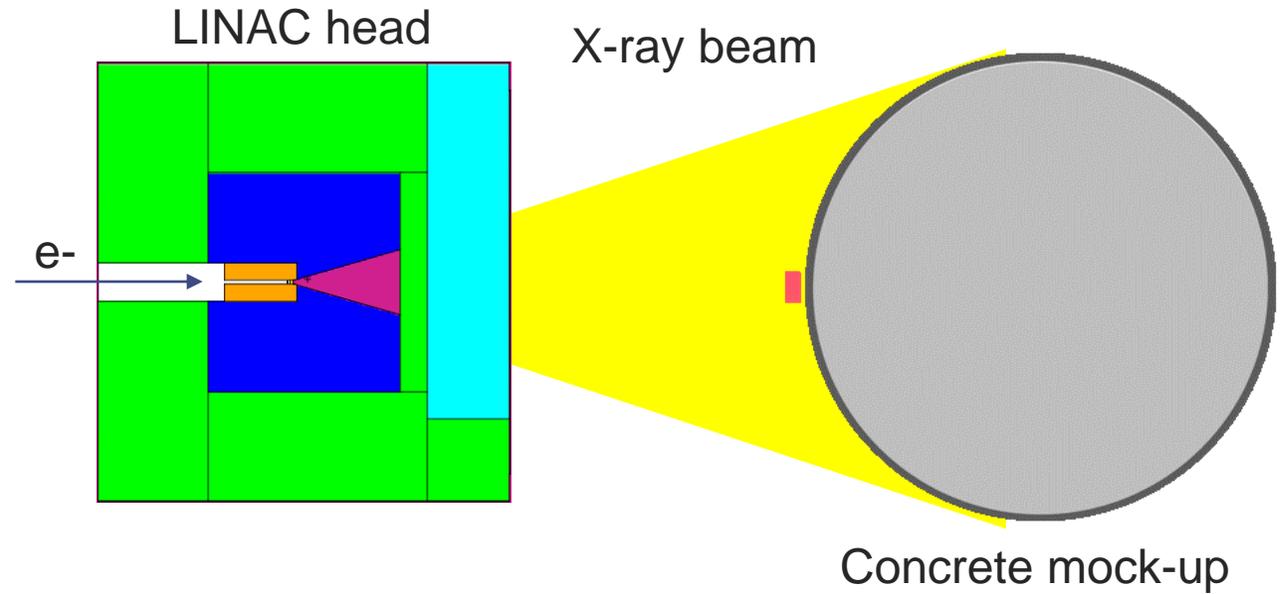
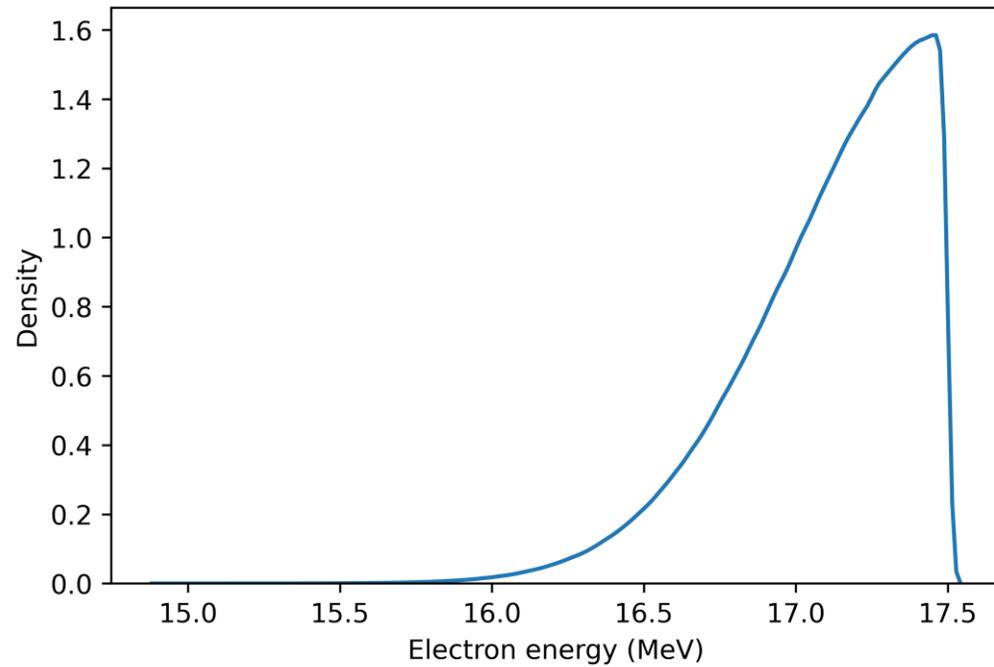


1 min measurements

Modelling of the experiments

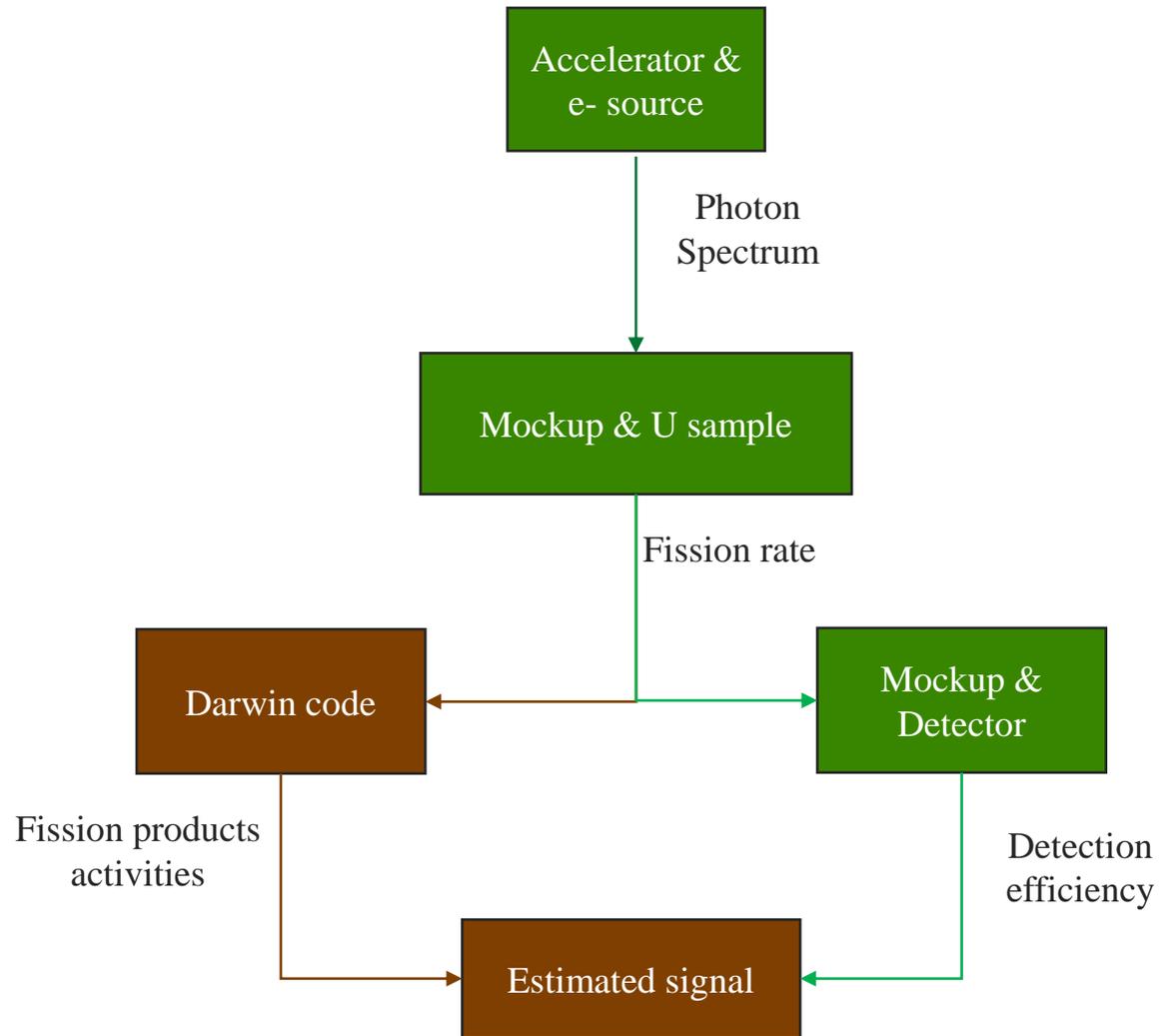


LINAC electron spectrum



LaBr₃ detector

Sequential method

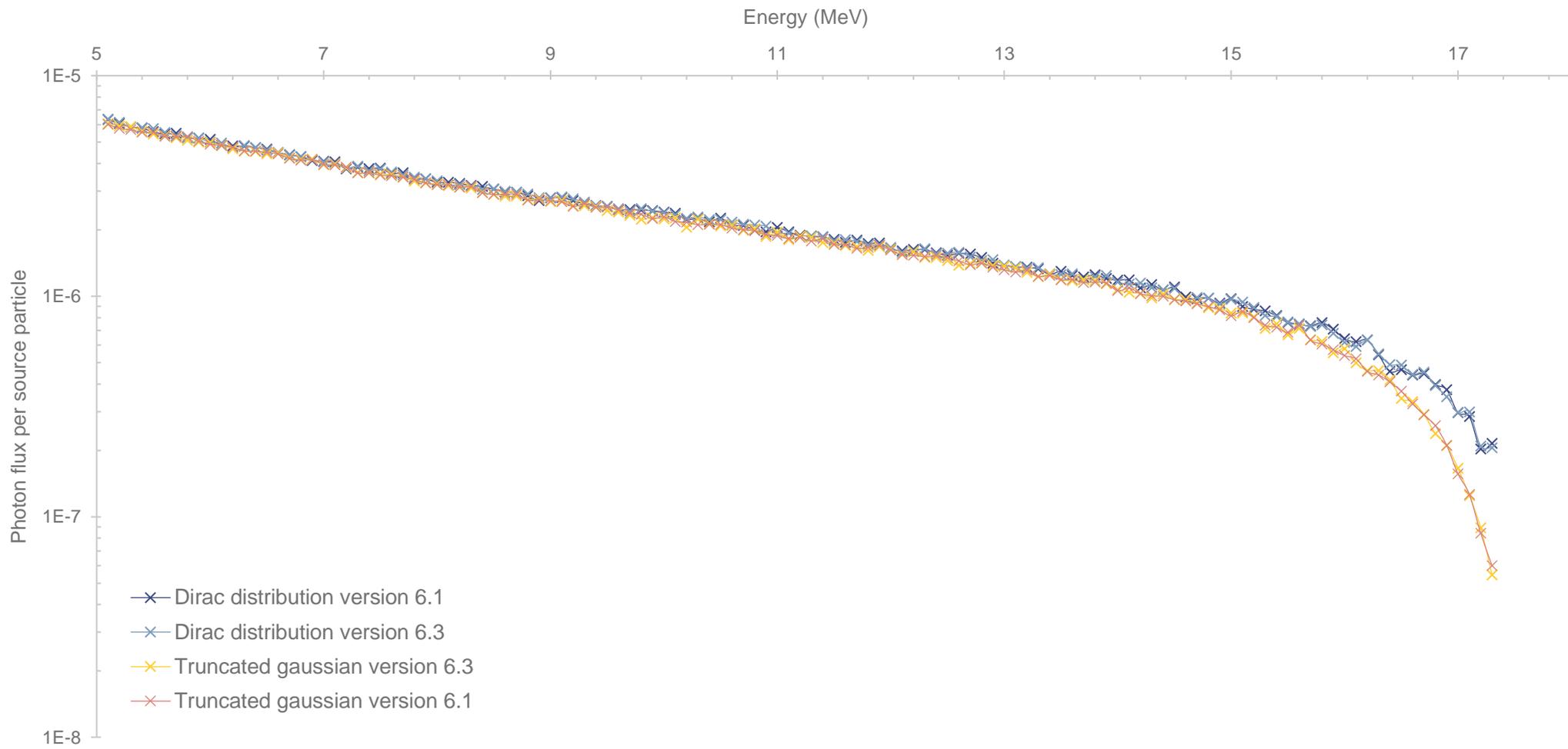


Finished work



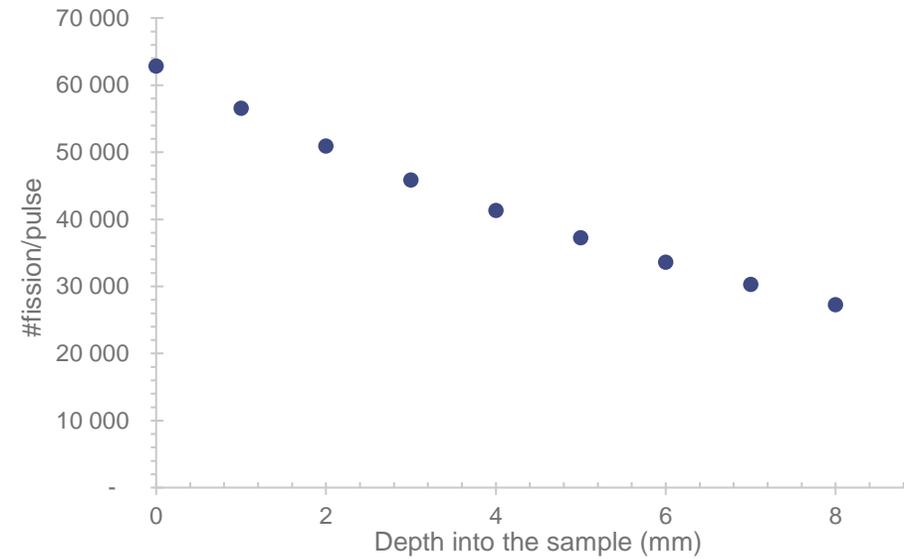
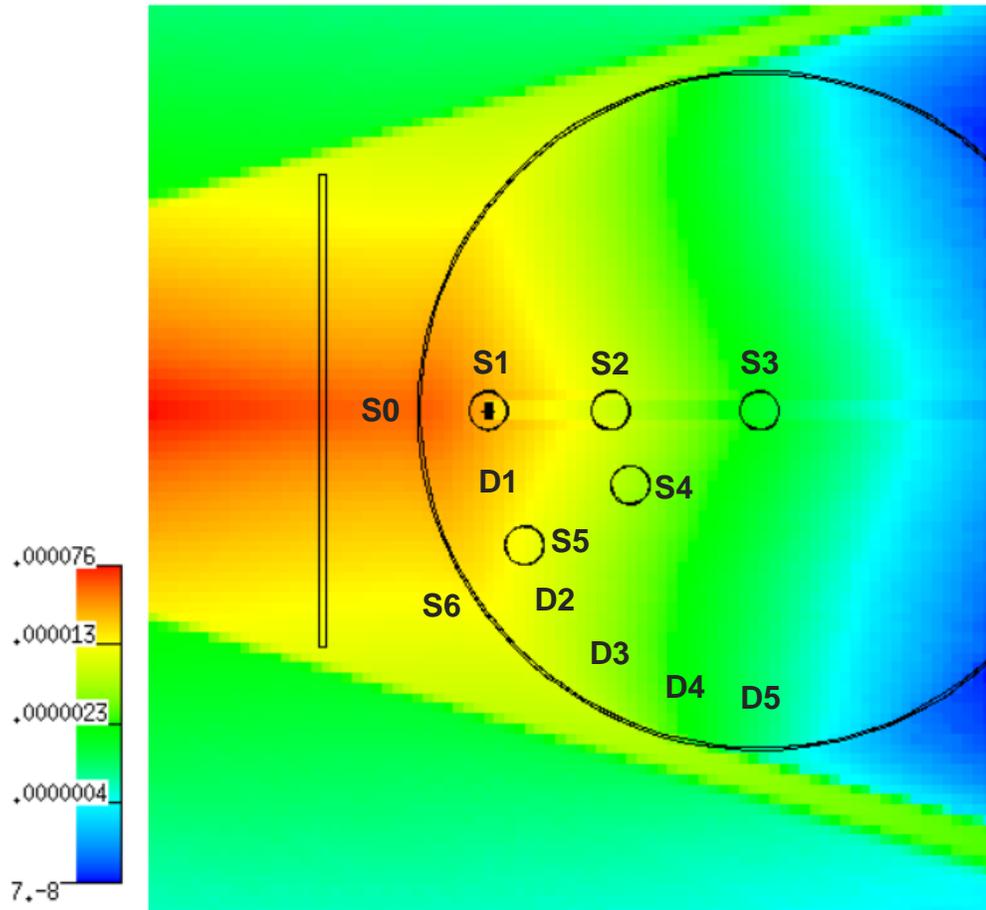
Planned work

1st step: Photon Spectrum



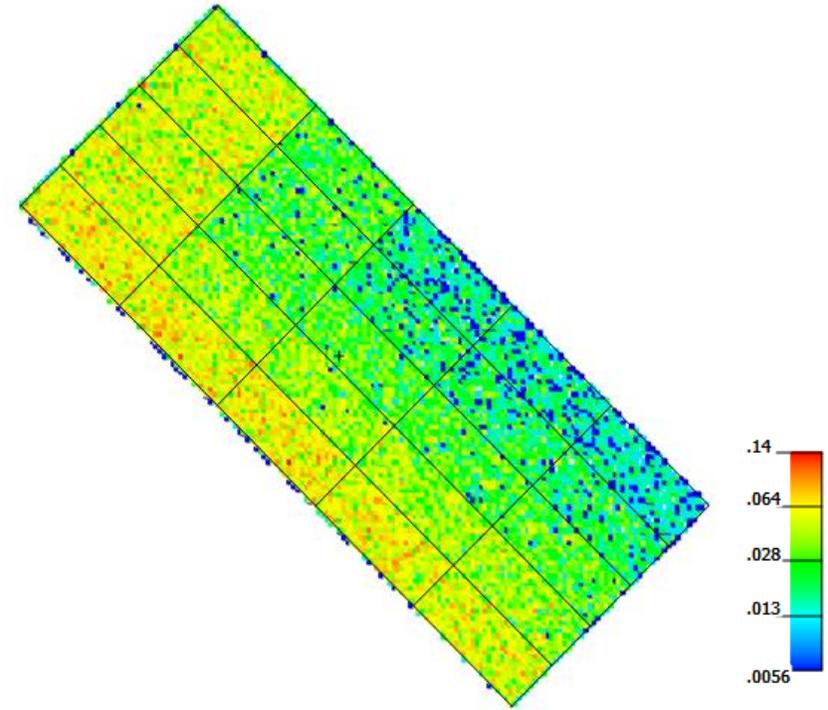
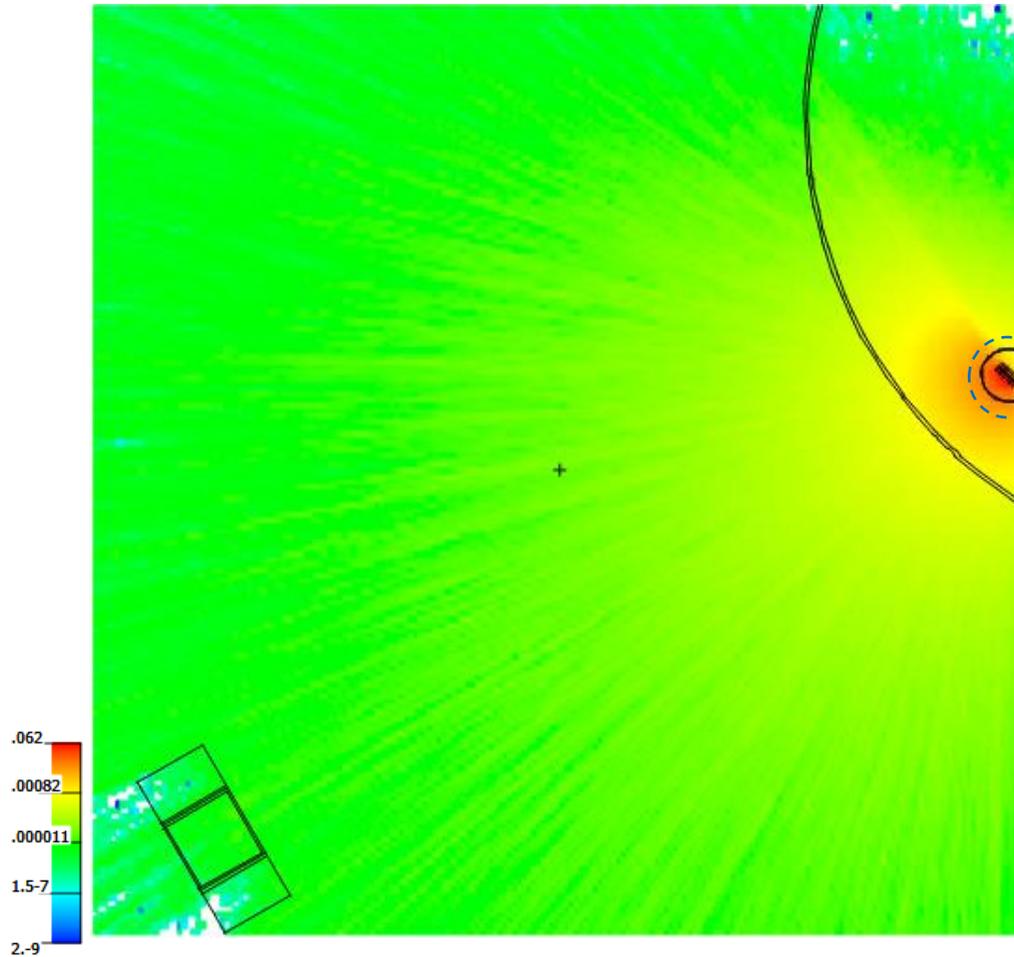
Source distribution	Dirac	Gaussian	Truncated Gaussian
Fission per e- source @ 30cm ($\times 10^{-7}$)	5.94	6.11	6.62

2nd step: Fission rate



Geometry	Depth (cm)	Angle (°)	#fission/pulse	Uncertainty
S0	0	0	749 650	0.1%
S1	7	0	410 350	0.1%
S2	25	0	136 300	0.3%
S3	50	0	34 800	0.2%
S4	25	30	69 600	0.3%
S5	7	30	120 350	1.3%
S6	0	30	162 400	0.5%
D1	7	15	227 650	0.4%
D2	7	45	62 350	0.7%
D3	7	60	30 450	1.5%
D4	7	75	14 500	2.3%
D5	7	90	8 700	3.7%

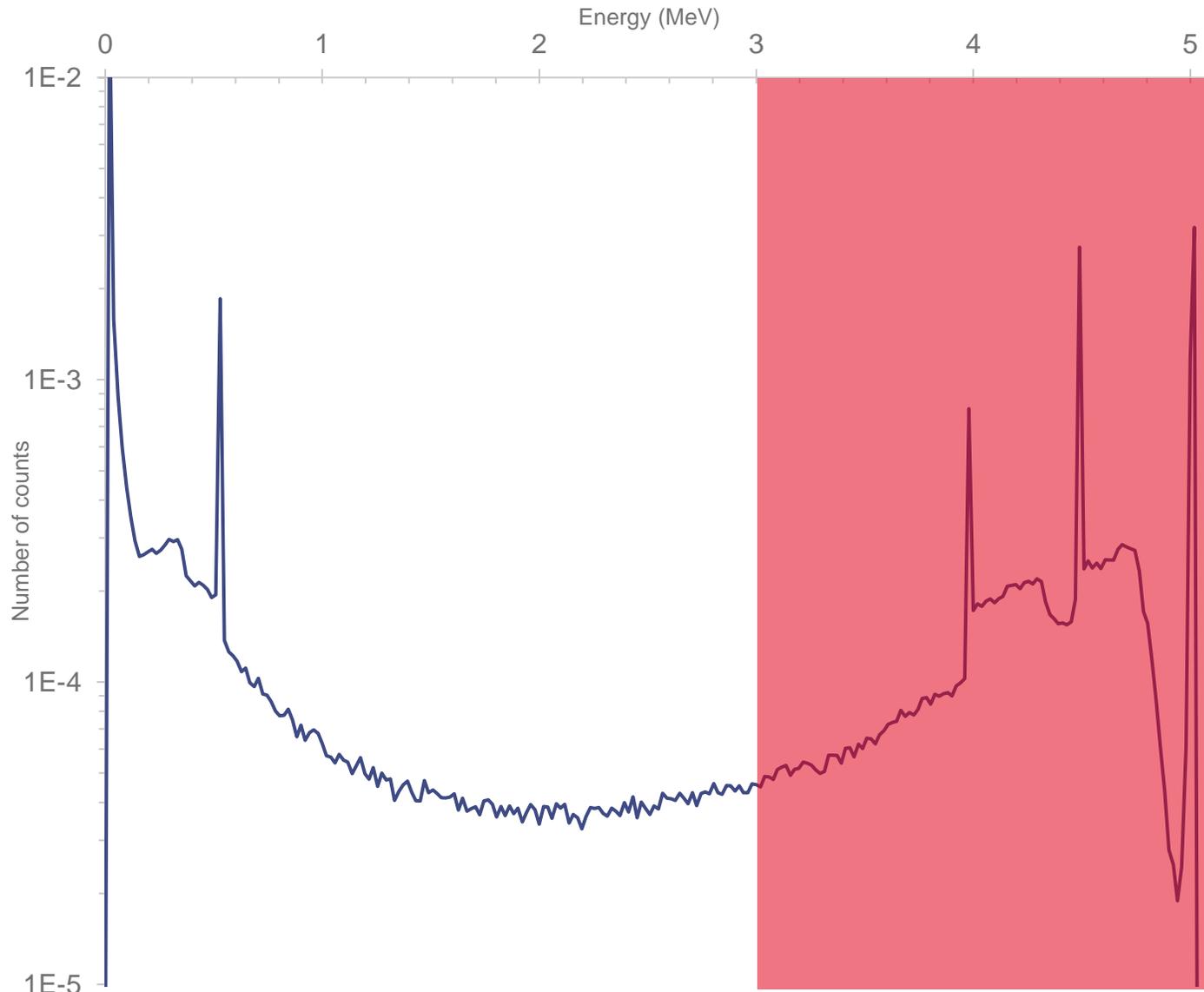
3rd step: detection efficiency



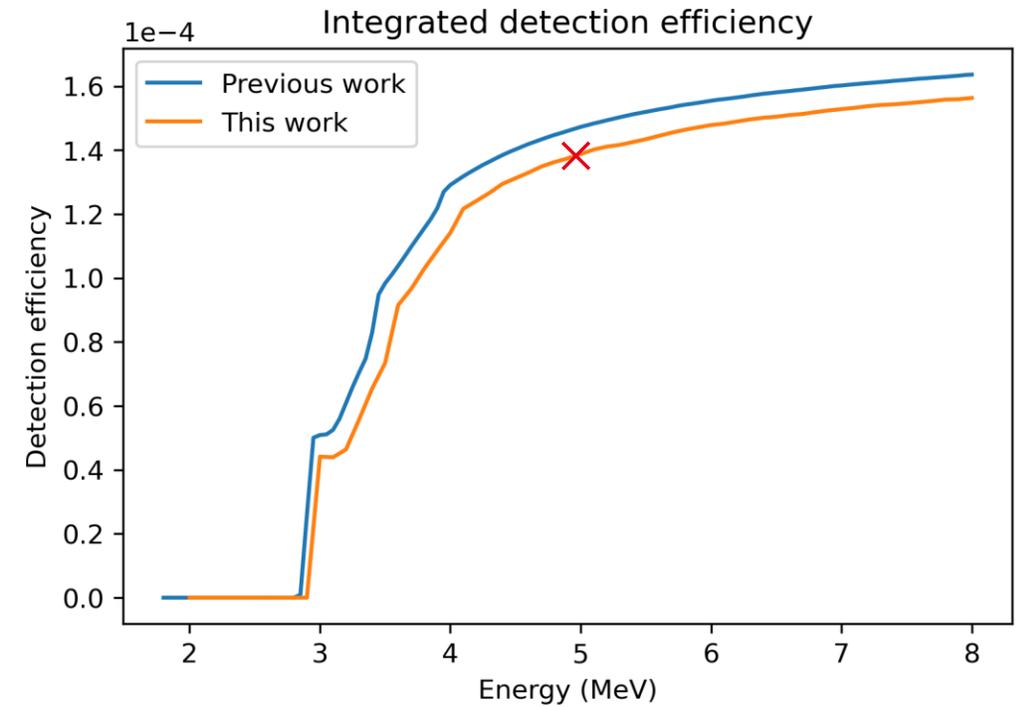
Uranium sample and gamma generation probability

Generate gamma for all energy between 2 MeV to 8 MeV with 100keV steps

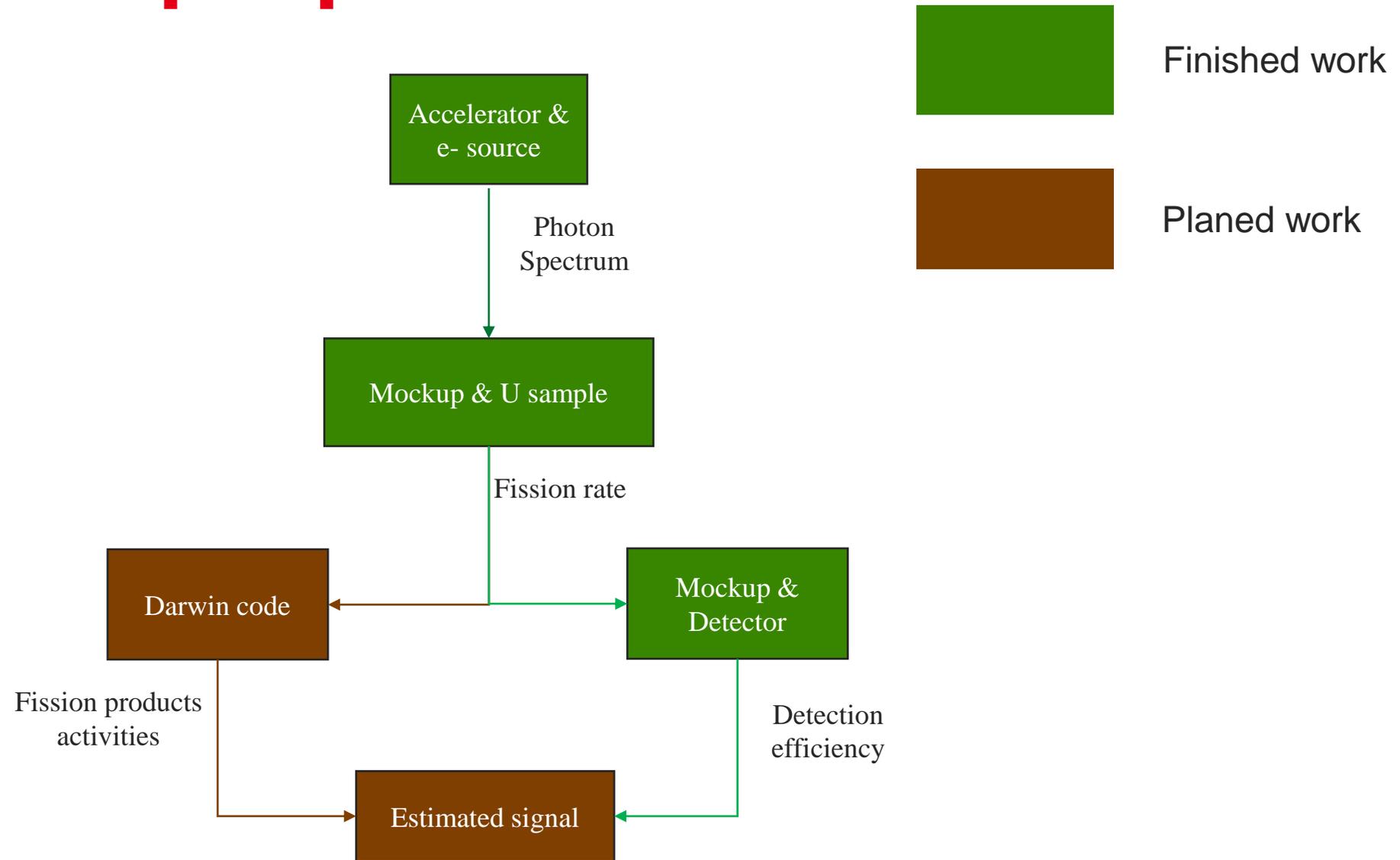
Detection efficiency: results



Detector signal, 5 MeV source, S3 geometry



Conclusion and prospects



Thanks for your attention !

