

# ${}^6\text{He}$ and ${}^{18}\text{Ne}$ production for beta-beams

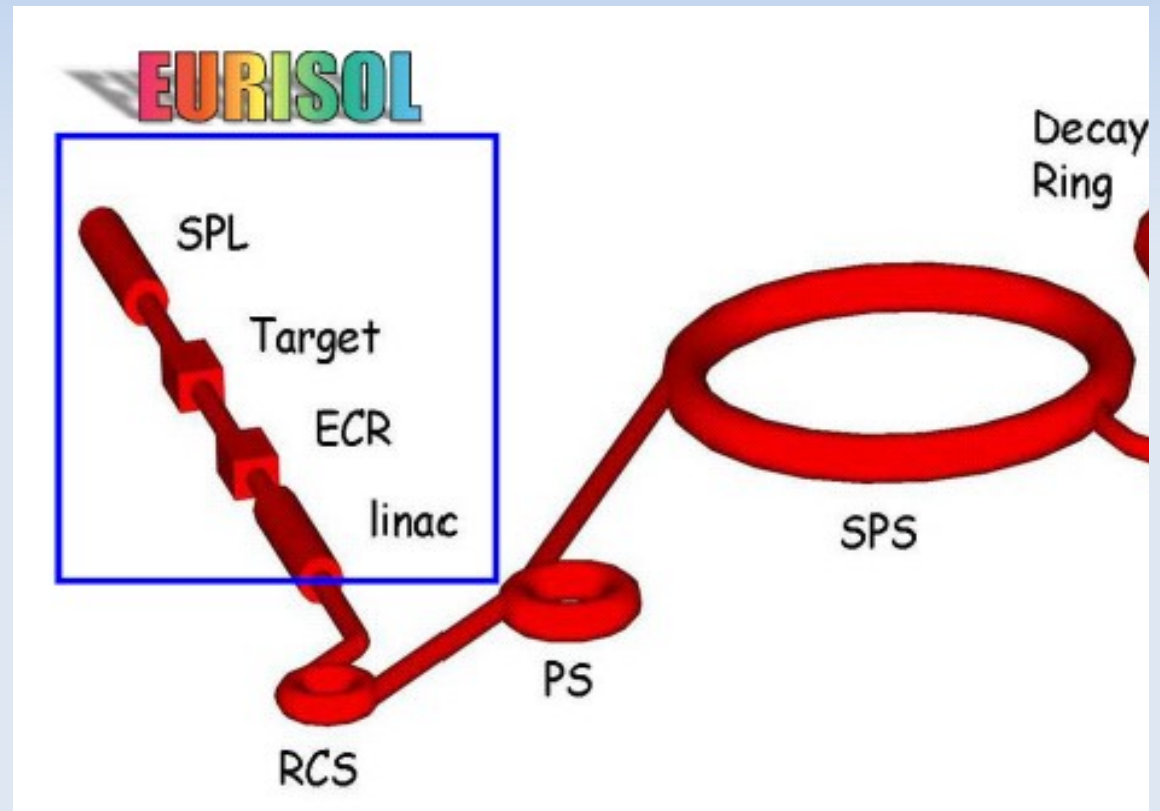
**Peter Valko**

**Thierry Stora**

Target and Ion Source Development  
EN-STI(ISOLDE)

For optimal sensitivity we need  
to deliver  
 $1.1 \cdot 10^{19} \nu$   
and  
 $2.9 \cdot 10^{19} \text{ anti-}\nu$   
over 10 years

Production of  $\nu_e$  &  $\text{anti-}\nu_e$ :  
 $3(.3) \cdot 10^{13} \text{ }^6\text{He/s}$   
 $2(.1) \cdot 10^{13} \text{ }^{18}\text{Ne/s}$   
**out of the primary target**  
(Final report, FP6 EURISOL-DS)

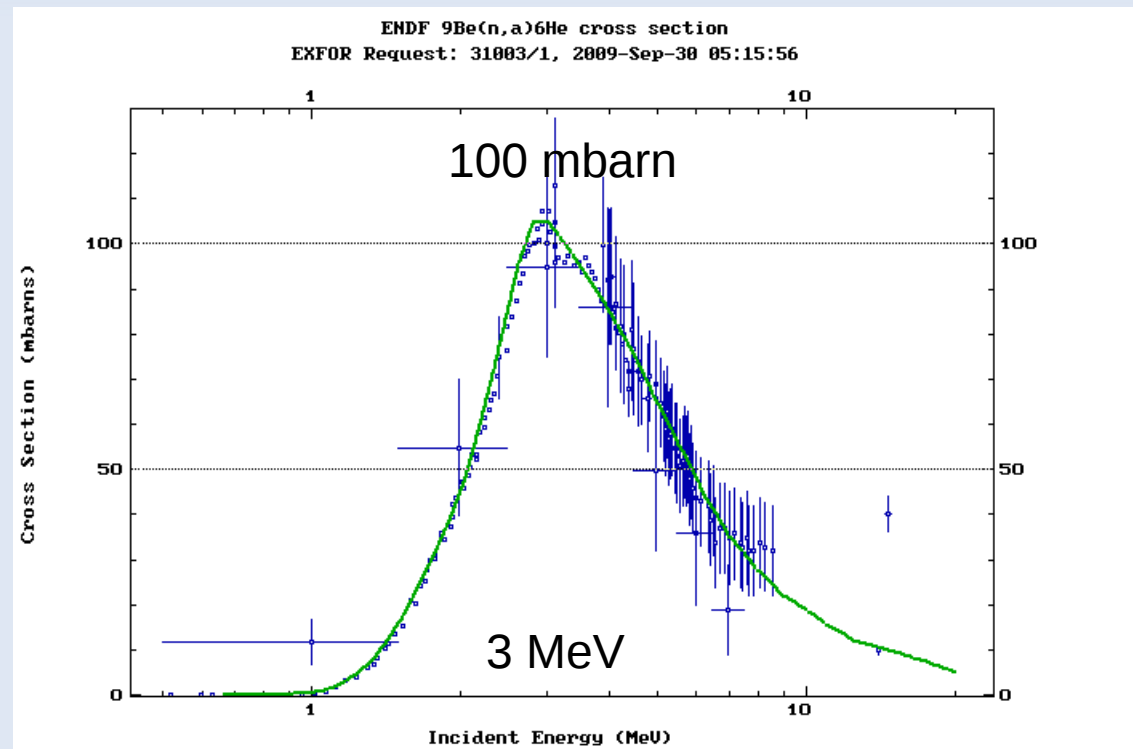


# Production of ${}^6\text{He}$ ions for beta beams

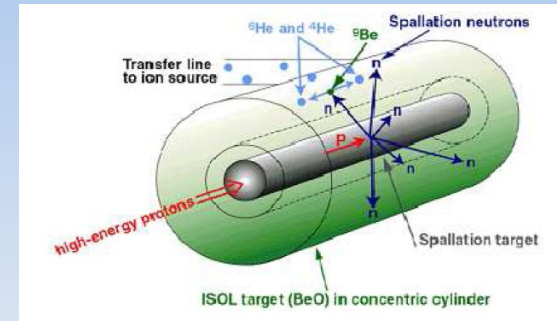
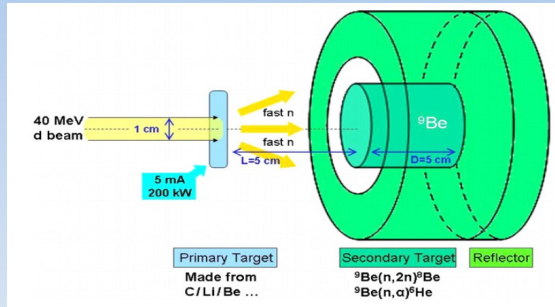
Elements driving the technical choices for isotopes and targets:

- Efficient production channels (high production cross-section  $\sigma$ )
- Isotopes “faciles” ( $t_{1/2}$ , release properties)
- Side effects (primary beam penetration range, heating, chemistry, ...)

Production of  $\nu_e$   
Out of the target  
 $3 \cdot 10^{13}$   ${}^6\text{He}/\text{s}$



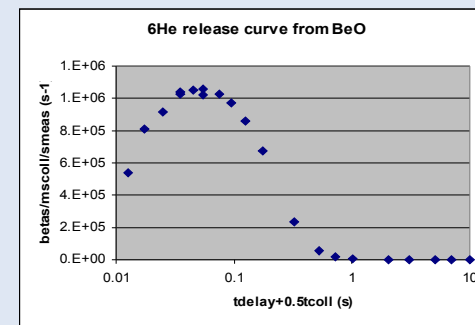
# Production of ${}^6\text{He}$ ions for beta beams



M. Hass et al

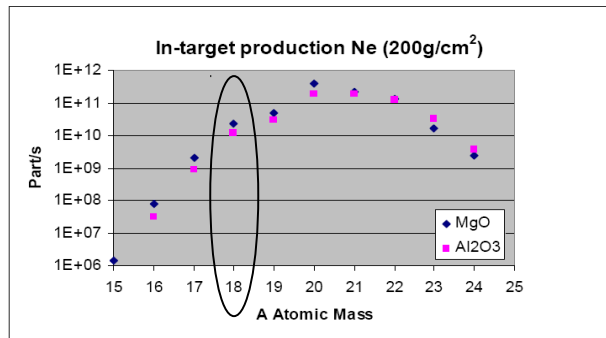
$10^{14}$   ${}^6\text{He}/\text{s}$  200 kW, 2 GeV proton beam in-target production. 50 – 90% extraction efficiency from realistic big targets (reduced scale experiment done at CERN-ISOLDE to confirm these numbers; Weizmann, GANIL and CERN collaboration).

Numbers have now been carefully checked. 1 kW of 1-2 GeV protons produces more  ${}^6\text{He}$  than 1 kW of 40 MeV deuterons.



# Production of $^{18}\text{Ne}$ ions for beta beams

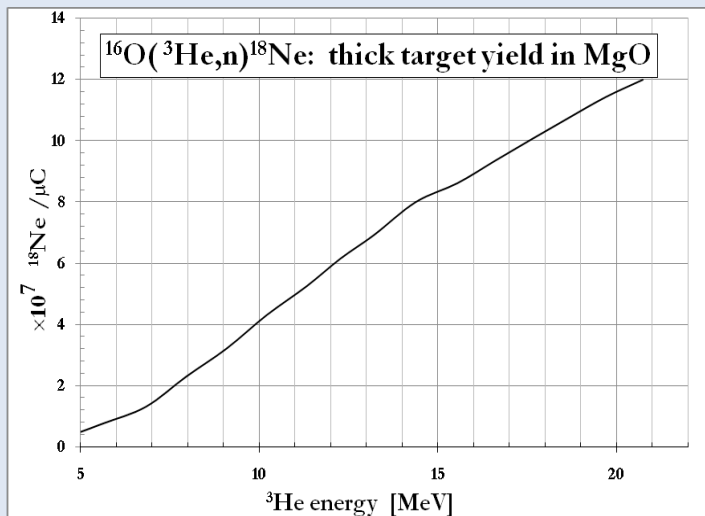
- Direct spallation of 1 GeV protons onto thick oxide targets Al (p,X)  $^{18}\text{Ne}$



Silberberg-Tsao,  
Thin target approx.

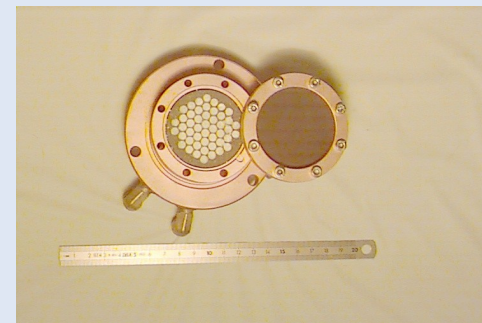
Nominal parameters:  
3 · 10<sup>10</sup> part/s (Fluka)

Production of  $\nu_e$   
Out of the target  
 $2 \cdot 10^{13}$   $^{18}\text{Ne}/\text{s}$



Validated at 9kW at LLN.  
Needs ~ 200mA,  $^3\text{He}$  21MeV, Ø86cm target

S. Mitrofanov,  
M. Loiselet et al.



# Production of $^{18}\text{Ne}$ ions for beta beams

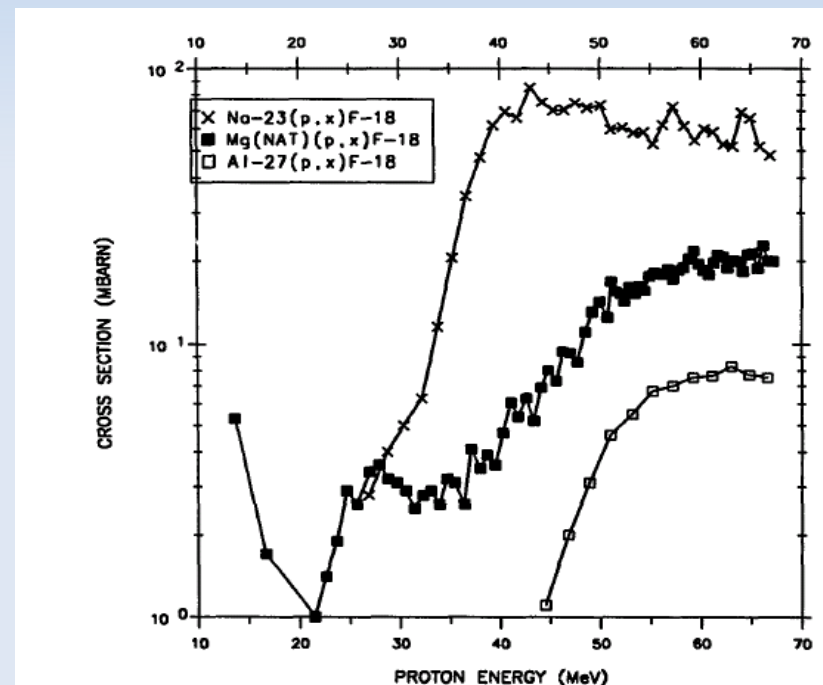
Other reactions (mainly coming from  $^{18}\text{F}$  production for PET imaging):

$^{19}\text{F}(p,2n)^{18}\text{Ne}$  : threshold 16MeV, peak at 1.6mbarn @ 30MeV (M. Loiselet, S. Mitrofanof)

$^{24}\text{Mg}(p,\alpha p2n)^{18}\text{Ne}$  : threshold 39 MeV, cross-sections ?

$^{27}\text{Al}(p,X)^{18}\text{Ne}$  :  $\sim 4$  mbarn @ 50-70 MeV (Lanulas-Solar, 1988&1992)

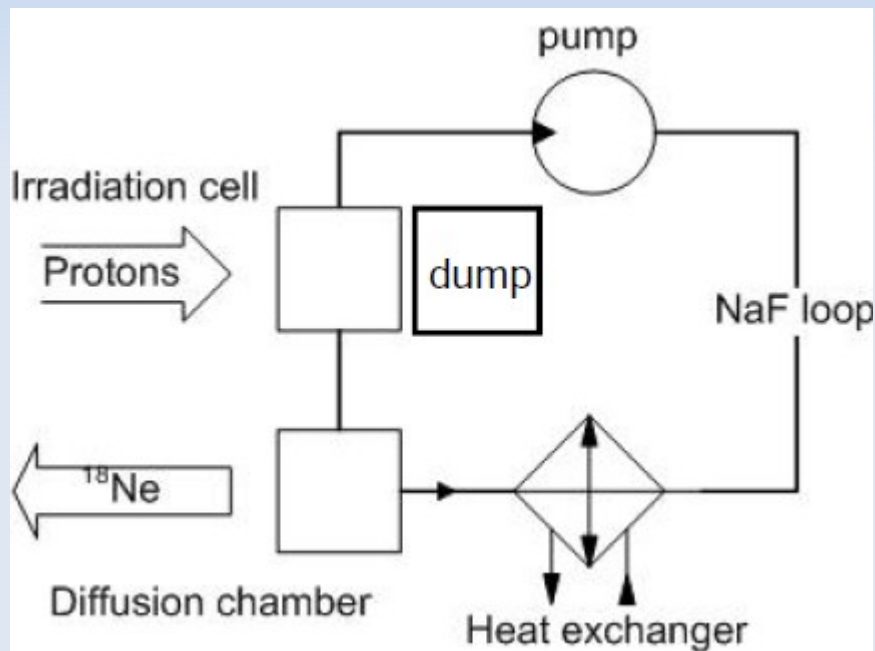
We need  $\sim 30\text{mA}$ ,  $70\text{MeV}$  p, and target R&D ( $\sim 600\text{kW}$  to be dissipated) for  $2 \cdot 10^{13}$   $^{18}\text{Ne}/\text{s}$



From workshop at CERN last year

# Production of $^{18}\text{Ne}$ ions for beta beams

Conceptual target loop for  $^{18}\text{Ne}$  production:



More than 300 reports on molten salt reactors available from ORNL  
Courtesy of O. Beneš, ITU

[Preface: Molten Salt Reactors](#)  
[Molten-Salt Reactors—History, Status, and Potential](#)  
[Experience with the Molten-Salt Reactor Experiment](#)  
[Molten-Salt Reactor Chemistry](#)  
[New Developments in Materials for Molten-Salt Reactors](#)  
[Engineering Development of the MSBR Fuel Elements](#)  
[Graphite and Reson Behavior and Their Influence on Molten-Salt Reactor Design](#)  
[The Design and Performance Features of a Single-Fluid Molten-Salt Breeder Reactor](#)  
[Breeder Physics and Fuel Cycle Analysis](#)

#### ORNL Reports Related to Liquid-Fluoride Reactors and Technology

ORNL-4962: 1999-09 ([848K PDF](#))  
Uses For Uranium-233: What Should Be Kept for Future Needs?

ORNL-5388: 1978-12 ([18.8M PDF](#))  
Interim Assessment of the Denatured  $^{235}\text{U}$  Fuel Cycle

ORNL-5176: 1977-02 ([6.2M PDF](#))  
Engineering Tests of the Metal Transfer Process from MSBR Fuel Salt

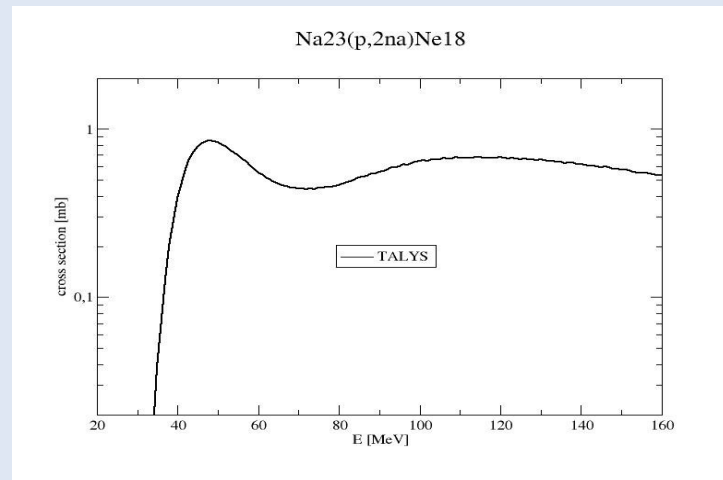
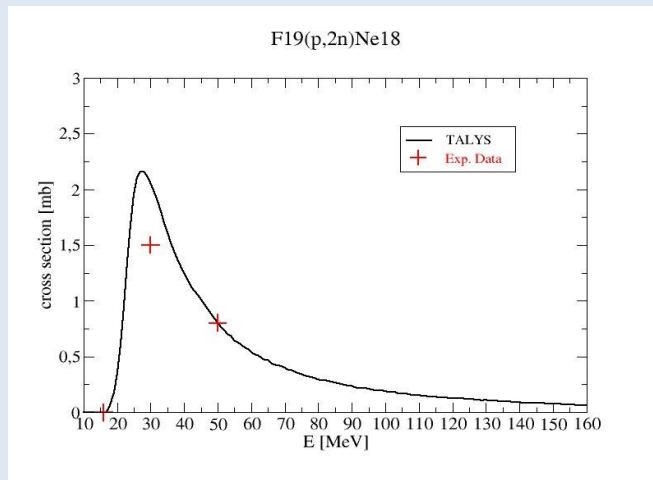
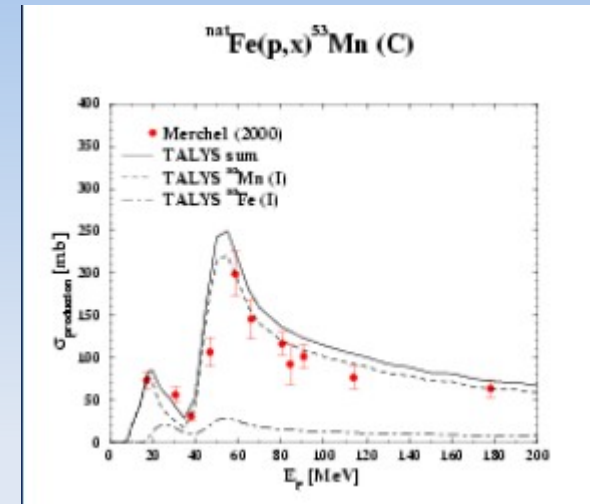
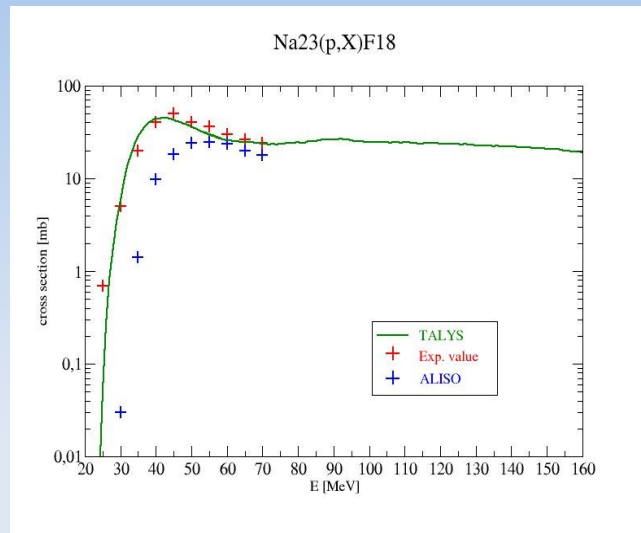
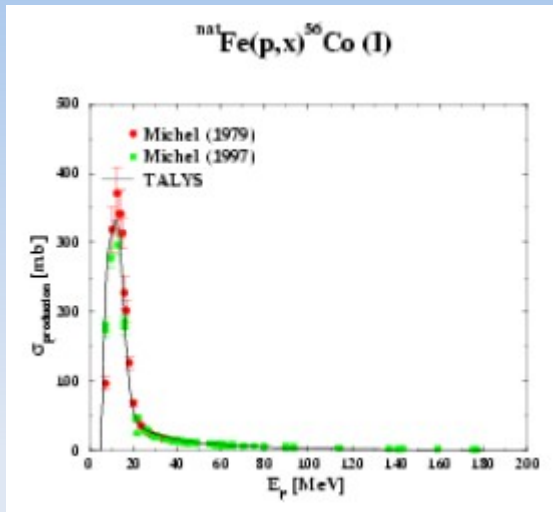
ORNL-5132: 1978-06 ([37.9M PDF](#))  
Molten-Salt Reactor Program: Semiannual Progress Report for Period Ending February 29, 1978

ORNL-5078: 1978-02 ([30.4M PDF](#))  
Molten-Salt Reactor Program: Semiannual Progress Report for Period Ending August 31, 1975

ORNL-5047: 1975-09 ([32.1M PDF](#))  
Molten-Salt Reactor Program: Semiannual Progress Report for Period Ending February 28, 1975

ORNL-5018: 1974-12 ([46.8M PDF](#))  
Program Plan for the Development of Molten-Salt Breeder Reactors

# TALYS code benchmarking



A.J. Koning, S. Hilaire, M.C. Duijvestijn, „TALYS-1.0“ Proceedings of the International Conference on Nuclear Data for Science and Technology, April 22-27, 2007, Nice, France, editors O. Bersillon, F. Gunsing, E. Bauge, R. Jacqmin, S. Leray, EDP Sciences, 2008, p. 211-214

M.C. Lagunas-Solar in Proc. of the IAEA consultants' meeting in data requirements for medical radioisotopes production, INDC(NDS)-195/GZ, 1988, p.55



# Production of $^{18}\text{Ne}$ ions for beta beams

Selection of a suitable eutectic comprising Na and F nuclei:

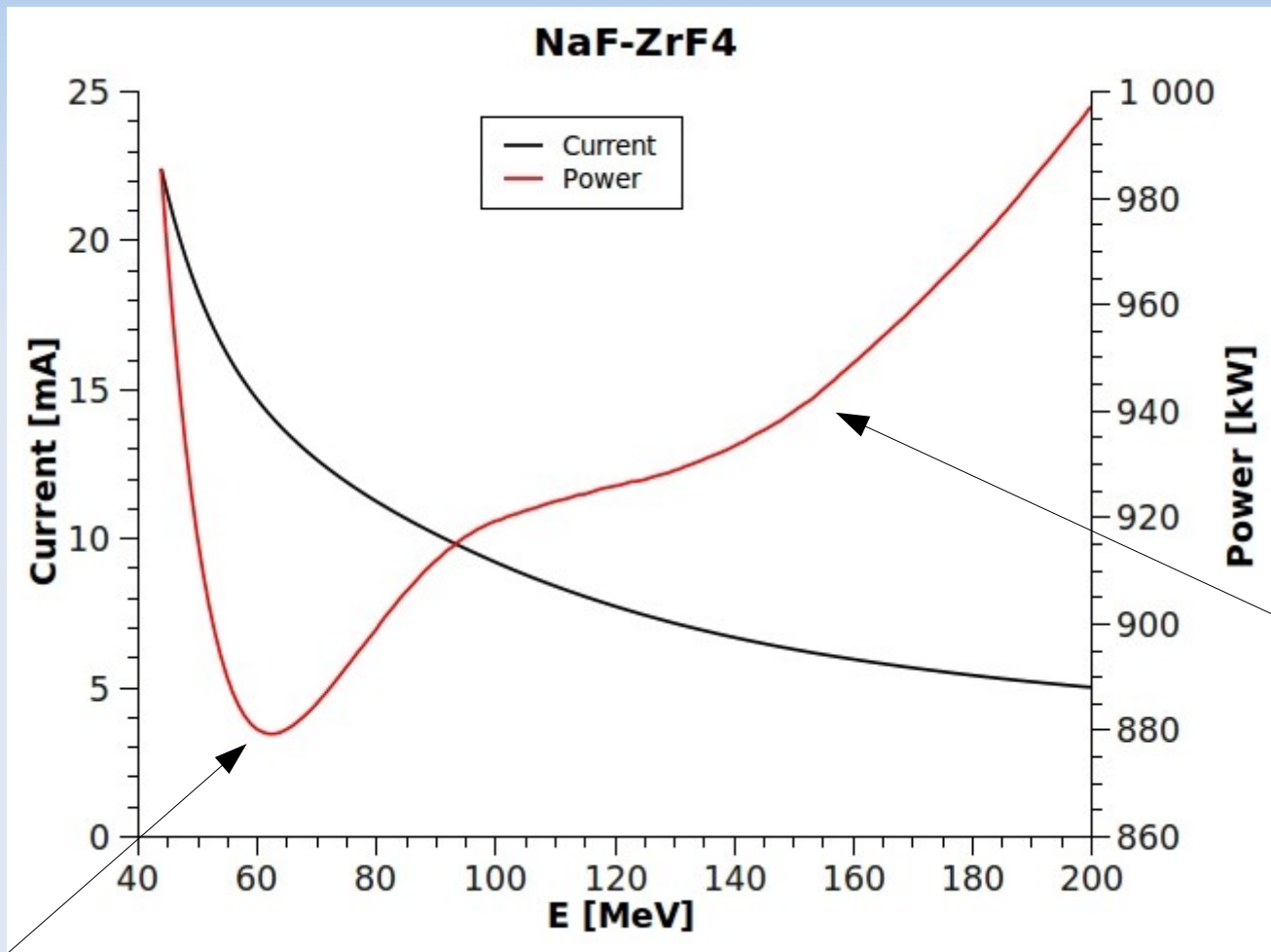
Melting point of NaF is ca 1000 °C

Salt	Composition [mol %]	Melting point [°C]	Density [g/cm <sup>3</sup> ] (700 °C)	Viscosity [cP] (700°C)	Vapor pressure [mmHg] (900°C)	Yield protons 6mA 160MeV	Yield helium3 6mA 160MeV
NaF-BeF <sub>2</sub>	57 - 43	340	2.01	7	1.4	8.8E+012	7.1E+012
NaF-NaBF <sub>4</sub>	8 - 92	384	1.75	0.9	9500	8.4E+012	6.9E+012
NaF-ZrF <sub>4</sub>	60 - 40	500	3.14	5.1	5	1.0E+013	8.2E+012

D.F. Williams, Assessment of Candidate Molten Salt Coolants for the NGNP/NHI Heat-Transfer loop, ORNL/TM-2006/69, Oak Ridge National Laboratory, Oak Ridge, TN (2006)

# Production of $^{18}\text{Ne}$ ions for beta beams

The beam current and power for a constant yield  $10^{13} \text{ }^{18}\text{Ne/s}$



Upgraded  
LINAC 4

Lowest power

# Production of $^{18}\text{Ne}$ ions for beta beams

## Candidate materials for molten salt loop

Candidate material	Salt corrosion resistance	Air corrosion resistance	Long-term strength at 1000 °C	Highest usage temperature [°C]
Hatelloy N	Excellent	Good	Very good	730
Haynes 242	Very good	Good	Very good	540
Alloy 800H or HT	Poor-fair	Good	Very good	980
Haynes 214	Very good	Good	Good	1000
MA 956	Very good	Good	Good	?
MA 754	Very good	Good	Good	?
Cast Ni superalloys	Very good	Good	Good	?

O. Benes, et. al., ALISA, Review Report on Liquid Salts for Various Application, version V4

# Production of $^{18}\text{Ne}$ ions for beta beams

## Candidate materials for molten salt loop

Hastelloy N is the most promising candidate for application up to 750 °C

Hastelloy N composition:

	Cr	Mo	W	Al	Ti	Fe	C	Co	Ni
Hastelloy N	6.31	16.1	0.06	0.01	4.03	0.03	0.03	0.15	72.2

O. Benes, et. al., ALISA, Review Report on Liquid Salts for Various Application, version V4

# Production of $^{18}\text{Ne}$ ions for beta beams

## Candidate materials for molten salt loop

Loop #	Alloy	Salt	Duration [h]	$T_{\text{max}}$ [°C]	Corrosion depth [mil]	ORNL report # or [reference]
15A	INOR-8	73LiF-2BeF <sub>2</sub> -25ThF <sub>4</sub>	39476	677	0.05	TM-4286
1208	INOR-8	FLiNaK	8760	677	1	2799
1190	INOR-8	58NaF-35BeF <sub>2</sub> -7ThF <sub>4</sub>	8760	677	1	2799
1233	INOR-8	71LiF-16BeF <sub>2</sub> -13ThF <sub>4</sub>	8760	677	0	2973
1213	INOR-8	71LiF-29ThF <sub>4</sub>	3114	677	0	2626
15	INOR-8	73LiF-2BeF <sub>2</sub> -25ThF <sub>4</sub>	2003	677	0	TM-4286
1165	INOR-8	FLiNaK	1340	677	0	2551
1164	INOR-8	58NaF-35BeF <sub>2</sub> -7ThF <sub>4</sub>	1000	677	0	2551
1221	INOR-8	71LiF-29ThF <sub>4</sub>	1000	677	0	2626
1228	INOR-8	71LiF-16BeF <sub>2</sub> -13ThF <sub>4</sub>	1000	677	0	2723
MSRE	INOR-8	67LiF-33BeF <sub>2</sub>	26000	649	0	TM-4174
9354-3	INOR-8	35LiF-27NaF-38BeF <sub>2</sub>	19942	649	0	3215
1194	INOR-8	FLiNaK	1000	607	0	2551
1195	INOR-8	35LiF-27NaF-28BeF <sub>2</sub>	1000	607	0	2551

D.F. Williams, Assessment of Candidate Molten Salt Coolants for the NGNP/NHI Heat-Transfer loop, ORNL/TM-2006/69, Oak Ridge National Laboratory, Oak Ridge, TN (2006)

# Production of $^{18}\text{Ne}$ ions for beta beams

## Scaling of the irradiation chamber:

For  $\Delta T = 100\text{ }^\circ\text{C}$ ,  $C_p = 1.17\text{ J/g.K}$ , we need **2.1 l/s**

Size of the window is  $360\text{ cm}^2$ ,  
e.g.  $15 \times 24\text{ cm}^2$   
Projected range is  $7.5\text{ cm}$

Size of the chamber:  
 **$15 \times 24 \times 7.5\text{ cm}^3$**

J.A. Lane, H.G. MacPherson, F. Maslan, Fluid Fuel Reactor, Chapter 13, Addison-Wesley, Reading, Mass. (1958)

Calculated with SRIM

Position on axis [cm]	Stopping power [MeV/cm]	Deposited beam power 6mA, 160MeV [kW]
1	12	74
2	13	78
3	14	83
4	15	90
5	16	99
6	19	111
7	22	133
8	30	178
Total target	126 MeV	756
Total dump	31 MeV	186
Total window (1 mm thick)	3 MeV	18

# Production of $^{18}\text{Ne}$ ions for beta beams

## Scaling of the irradiation chamber:

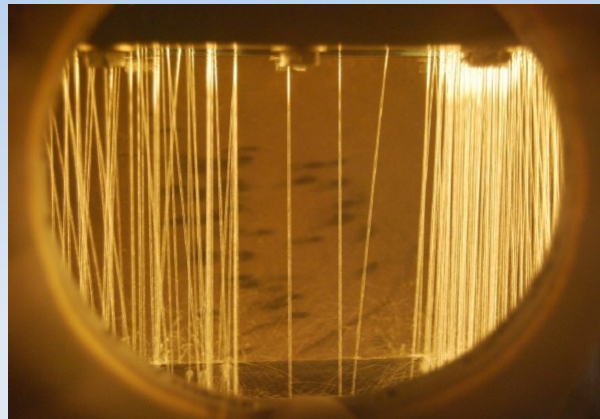
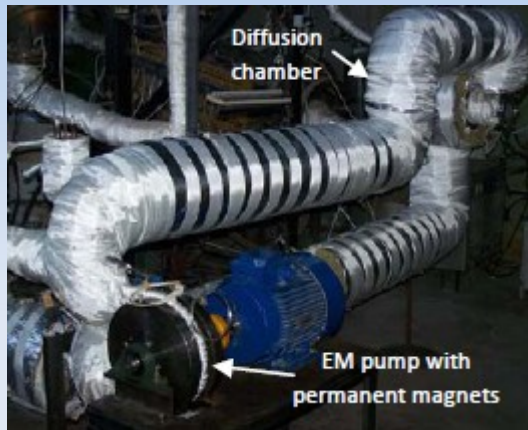
Cooling of the window (10 kW for 0.5 mm thickness) is done with the circulating molten salt.

Heat transfer coefficient estimates are available on salt/Hastelloy N

Salt	Reynolds modulus	Prandtl modulus	Nusselt modulus	Fluid temperature [°C]	Heat transfer coefficient [ $\text{W}\cdot\text{m}^{-2}\cdot\text{K}^{-1}$ ]
NaF-NaBF <sub>4</sub>	5100-45000	5,3-5,64	35-225	450-610	1380-10100

# Production of $^{18}\text{Ne}$ ions for beta beams

## Scaling of the diffusion chamber:



E. Noah @ IPUL Lab  
Molten Pb/Bi  
EURISOL DS

M. Fujioka, Y. Arai, Diffusion of Radioisotopes from Solids in the form of Foils, Fibers and Particles, Nucl. Instr. and Meth. 186 (1981) 409

Diffusion coefficients

$D = 2-4 \cdot 10^{-5} \text{ cm}^2/\text{s}$  for Kr and Xe

D estimate at  $4-8 \cdot 10^{-5} \text{ cm}^2/\text{s}$  for Ne

Mean diff. Time 0.13 s for  $5 \cdot 10^{-5} \text{ cm}^2/\text{s}$

R.J. Kedl, A. Houtzeel, ORNL-4069: 1967-06

Diffusion coefficient [mm <sup>2</sup> .s <sup>-1</sup> ]	Hole radius [mm]	Released fraction Cylinder	Released fraction Sphere
1.0E-003	0.25	0.35	0.47
2.5E-003	0.25	0.5	0.63
5.0E-003	0.25	0.64	0.76
1.0E-003	0.1	0.68	0.79
2.5E-003	0.1	0.83	0.9
5.0E-003	0.1	0.91	0.95



# Production of $^{18}\text{Ne}$ ions for beta beams

## Scaling of the diffusion chamber

Volume in diffusion chamber (droplets at 1 m/s)

Height: **40 cm** / tortuosity factor

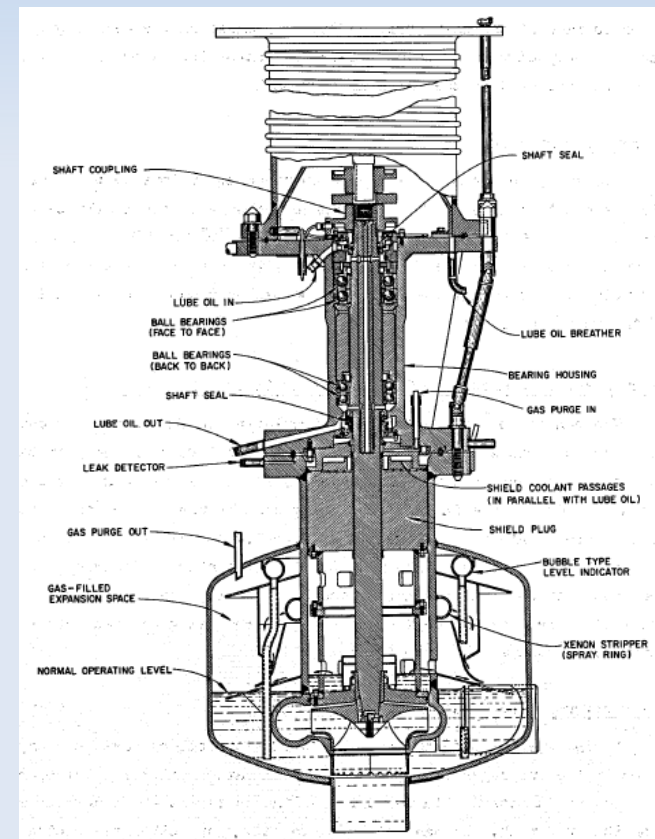
Surface: **15x15 cm<sup>2</sup>** / hole surface density (10 %)

# Production of $^{18}\text{Ne}$ ions for beta beams

## Pumps:

### Cross section of Mark-2 Fuel-Salt Pump

Model	Fluid <sup>d</sup>	Head (ft)	Flow (gal/min)	Speed (rpm)	Temperature (°F)	Number built	Total hours
LFB	Na, NaK, and MS	92	5	6000	1100–1400	46	466,000
DANA	Na, NaK, and MS	300	150	3750	1000–1500	10	57,000
DAC	MS	50	60	1450	1000–1400	3	4,000
In-Pile Loop	MS	10	1	3000		8	14,000
MF	NaK and MS	50	700	3000	1100–1500	3	41,000
PKA	NaK and MS	400	375	3550	700–1500	2	21,500
PKP	NaK and MS	380	1500	3500	700–1500	4	45,000
MSRE fuel salt pump	MS and Helium	50	1200	1175	1000–1225 100–1200	2	31,600 6,000
MSRE coolant salt pump	MS and Helium	78	800	1775	1000–1225 100–1200	2	24,600 4,000
MSRE Mark-2 fuel salt pump	MS	50	1200	1175	1000–1300	1	14,000
ALPHA	MS	300	30	6500	850–1400	1	6,000



P.G. Smith, ORNL/TM-2987 TN (1970)

C.W. Forsberg, ORNL/TM-2006/92

# Outlook

The goal is to provide a proposal with as close as possible technologies which are realistic. In particular, 100's kW rather than MW target dimensioning.

From present 1<sup>st</sup> order estimates, we need 6 mA, 160 MeV proton Linac, 700kW target loop technology, and ~ 8 years of operation to deliver the integrated  $1.1e19$   $\nu$ .

This will certainly still evolve. Some experimental data are absolutely required.

# Acknowledgements

E. Noah during EURISOL-DS for the molten metal loop prototype at IPUL

O. Beneš et al., ITU, JRC

E. Wildner, V. Vlachoudis, J. Lettry, R. Garoby, M. Vretenar, ...

Thank you!