

# Chemical decontamination of metallic waste and measurement of radionuclides

Tomo Suzuki-Muresan

A. Rivonkar, M. Robin, I. Moschetti, N. Bessagnet, O. Ileri, E. Abed,  
M. Mokili, A. Abdelouas, G. Montavon, A.-L. Nivesse



# General introduction

# Decommissioning waste



## Solid

- Steel
- Aluminium
- Concrete
- Graphite
- Resins
- ...



## Liquid

- Aqueous
- Solvents
- Complexants
- Oil
- LSC



## Emitters

- Alpha
- Beta
- Gamma

# Context, issues, challenges

## ❑ Multiple **origins** of metallic wastes

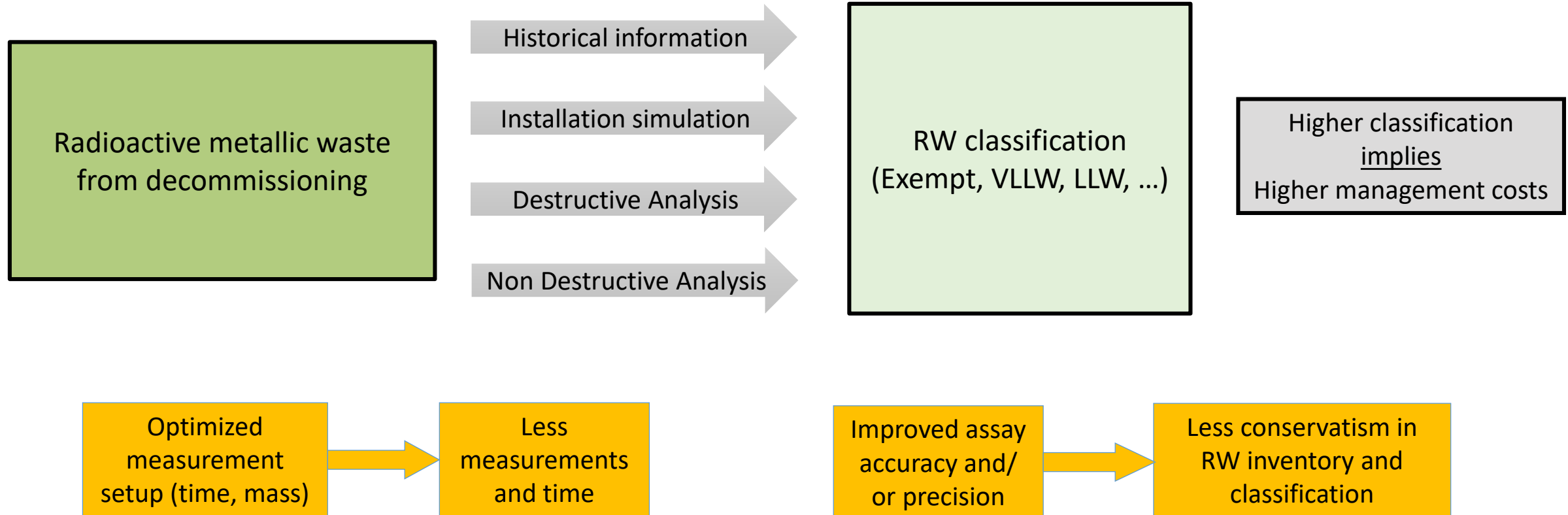
- Nuclear power plants: maintenance, dismantling, decommissioning
- Nuclear facilities for retreatment and reprocessing

## ❑ Dismantling and decommissioning

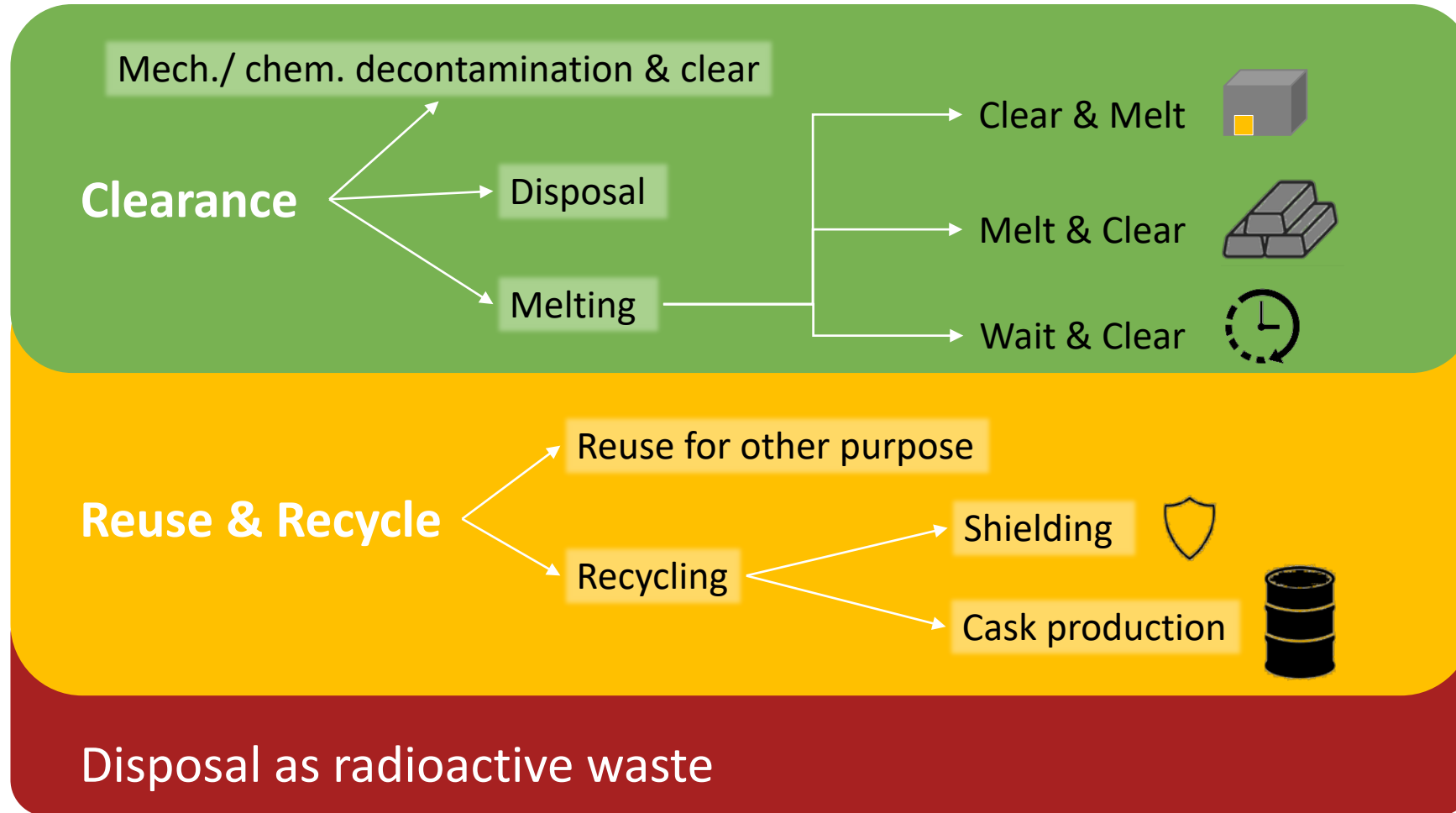
- Very **large volumes** of waste: metallic components one of the main contributor of the inventory  
⇒ may over-saturate the capacity of waste disposal repository.
- About 500 000 tons of metallic wastes expected
- Including about 130 000 tons from steam generators



# Necessity for optimizing metallic waste characterization & procedures for minimization and recycling



# Metallic Waste Management



# Decontamination

## ❑ Characterization

- oxidized metal matrices, radiochemical contamination

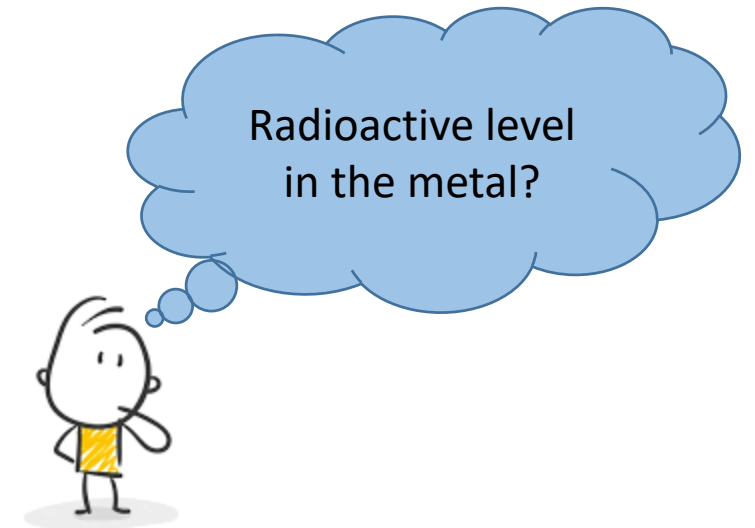
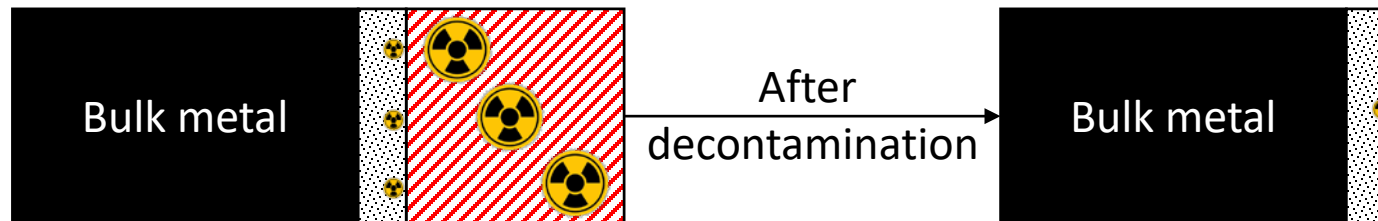
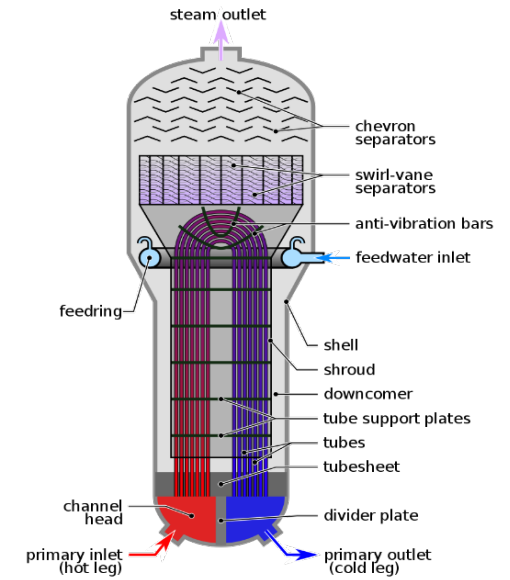
## ❑ Decontamination methods:

- chemical, mechanical, thermal, etc.

## ❑ Taking into account

- waste acceptance criteria from the waste management agency prior to decontamination
- difficulty of certain matrices to be decontaminated: e.g. tubes vs. pumps, pipe obstructions, etc.

## ❑ Industrial conditions: segmentation, fusion, in situ or ex situ



# Characterization & Measurements



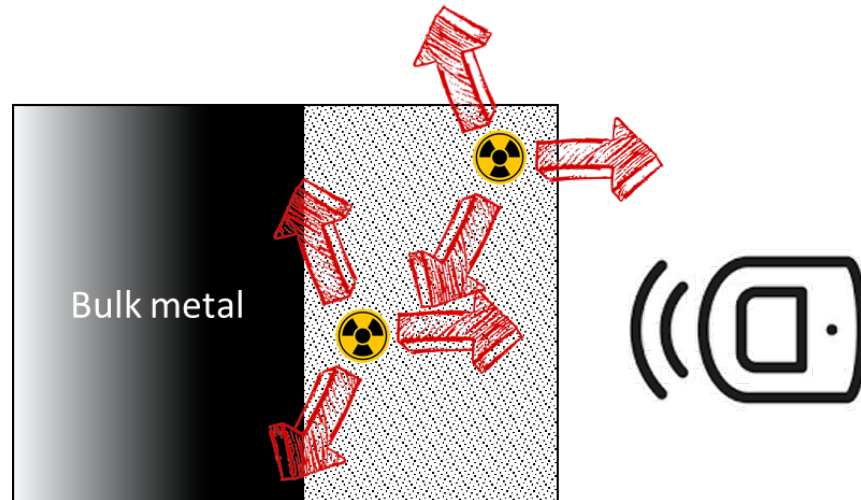
Measurements/characterization of decontaminated metal waste



Radioactivity level < release threshold ?



Radioactivity level of radionuclides difficult to measure



Radionuclide	Activity (kBq/cm <sup>2</sup> ) [1]
<sup>60</sup> Co	250-1000
<sup>54</sup> Mn	4-30
<sup>58</sup> Co	150-500
<sup>57</sup> Co	0-2
<sup>65</sup> Zn	2-6

Radionuclide	release threshold (solid) [2]
<sup>14</sup> C	1 Bq/g
<sup>36</sup> Cl	1 Bq/g
<sup>54</sup> Mn	0.1 Bq/g
<sup>55</sup> Fe	1000 Bq/g
<sup>58</sup> Co	1 Bq/g
<sup>60</sup> Co	0.1 Bq/g
<sup>63</sup> Ni	100 Bq/g
<sup>65</sup> Zn	0.1 Bq/g
<sup>93</sup> Zr	10 Bq/g
<sup>106</sup> Ru	0.1 Bq/g
<sup>125</sup> Sb	0.1 Bq/g
<sup>129</sup> I	0.01 Bq/g

[1] M.E. Pick, 1989, Decontamination and decommissioning of nuclear facilities, IAEA

[2] Décret 2013/59/EURATOM



# Waste acceptance criteria



## ❑ Radiological parameters

- Total activity level, dose rate, half-life, type of emitters
- Avoid fissile materials (guarantee under-critic conditions)

## ❑ Chemical parameters: stabilization of the waste matrices

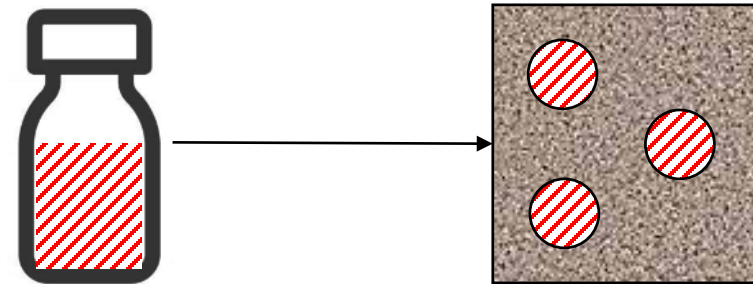
- Attention on the chemical reactivity, avoid metallic reactive wastes
- Limitations on complexing agents, accelerators of leaching processes (Cl, F...), organic substances (EDTA, NTA...), pyrophoric, flammable, explosive, corrosive or oxidizing
- Toxic chemical species are controlled
- Avoid liquid form, biologically active waste are forbidden

## ❑ Mechanical parameters:

- Compression resistance, void limitation, swelling
- Confinement of RN to limit the diffusivity and leachability

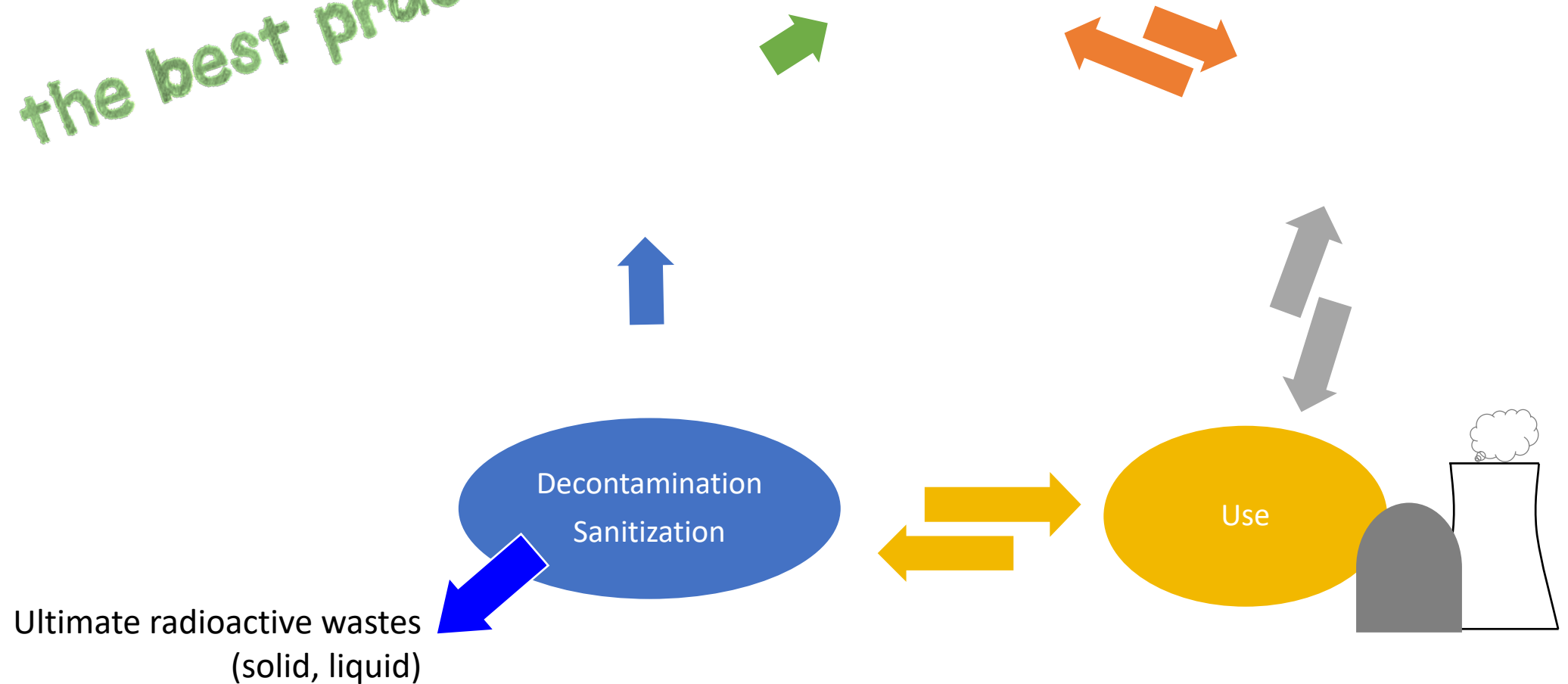
## ❑ Other parameters

- H<sub>2</sub> production, integrity and homogeneity of the waste form

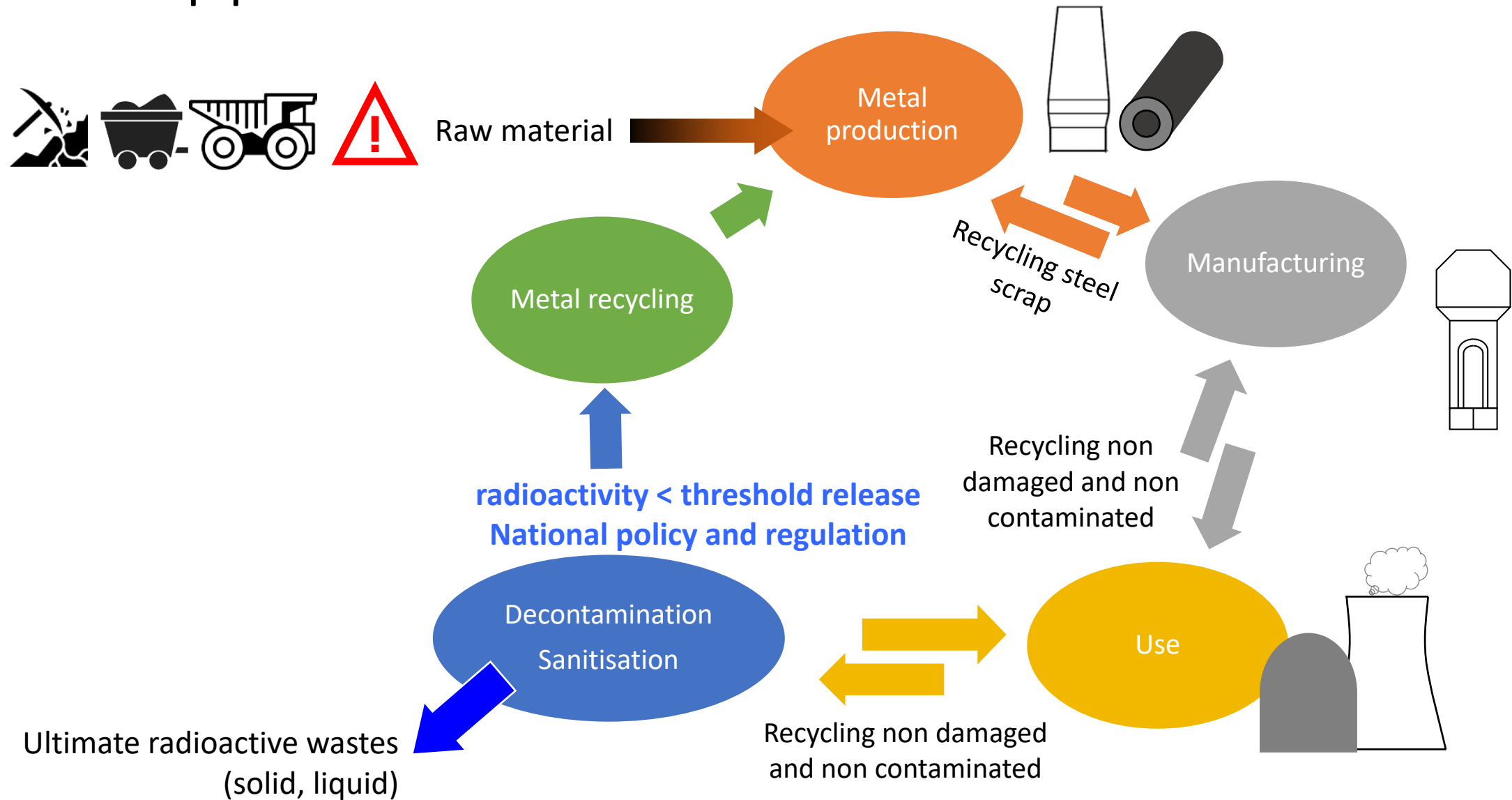


# Life Cycle Assessment (LCA) and Life Cycle Costing (LCC)

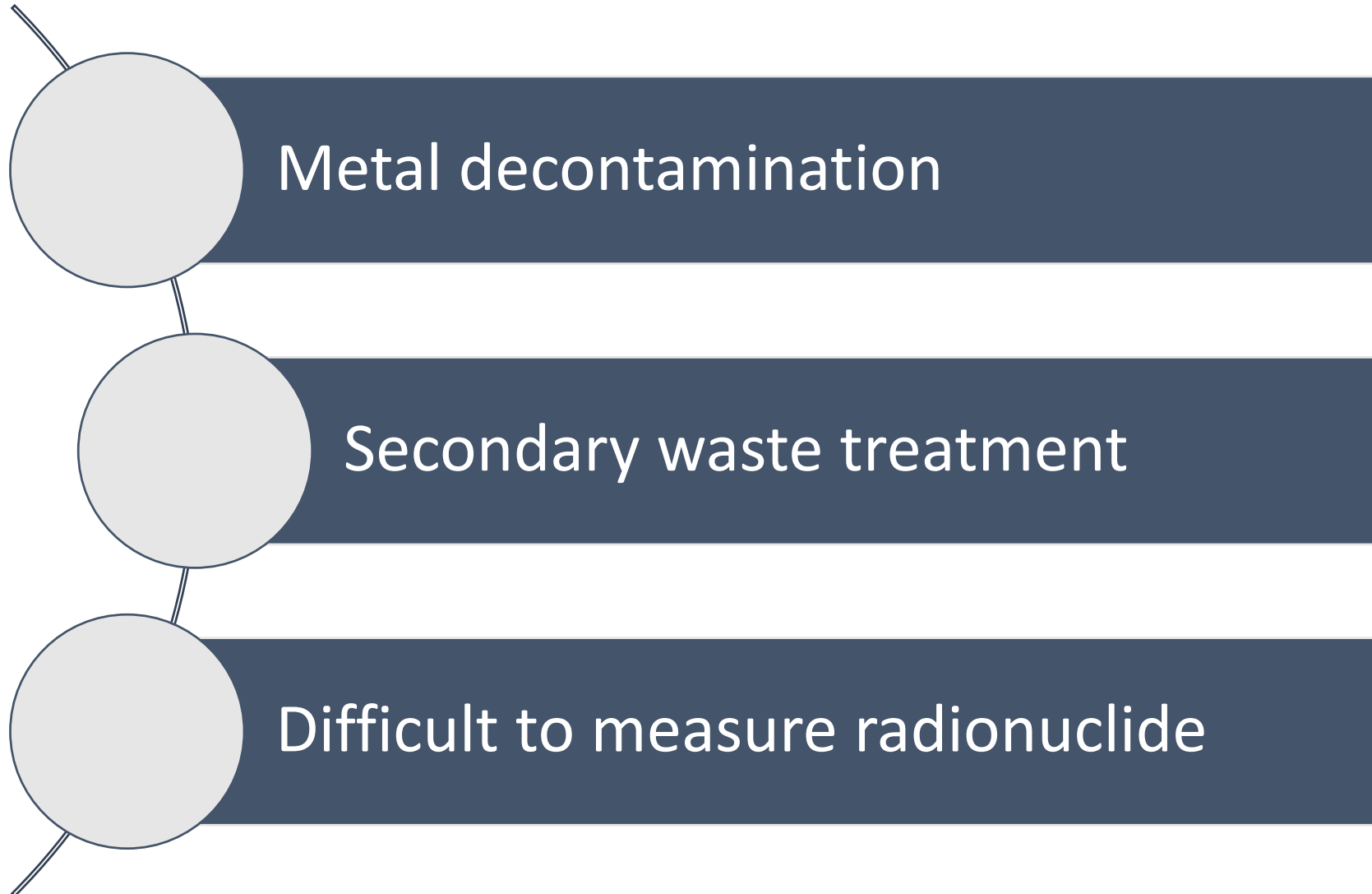
*Select the best practices*



# LCC / LCA approach

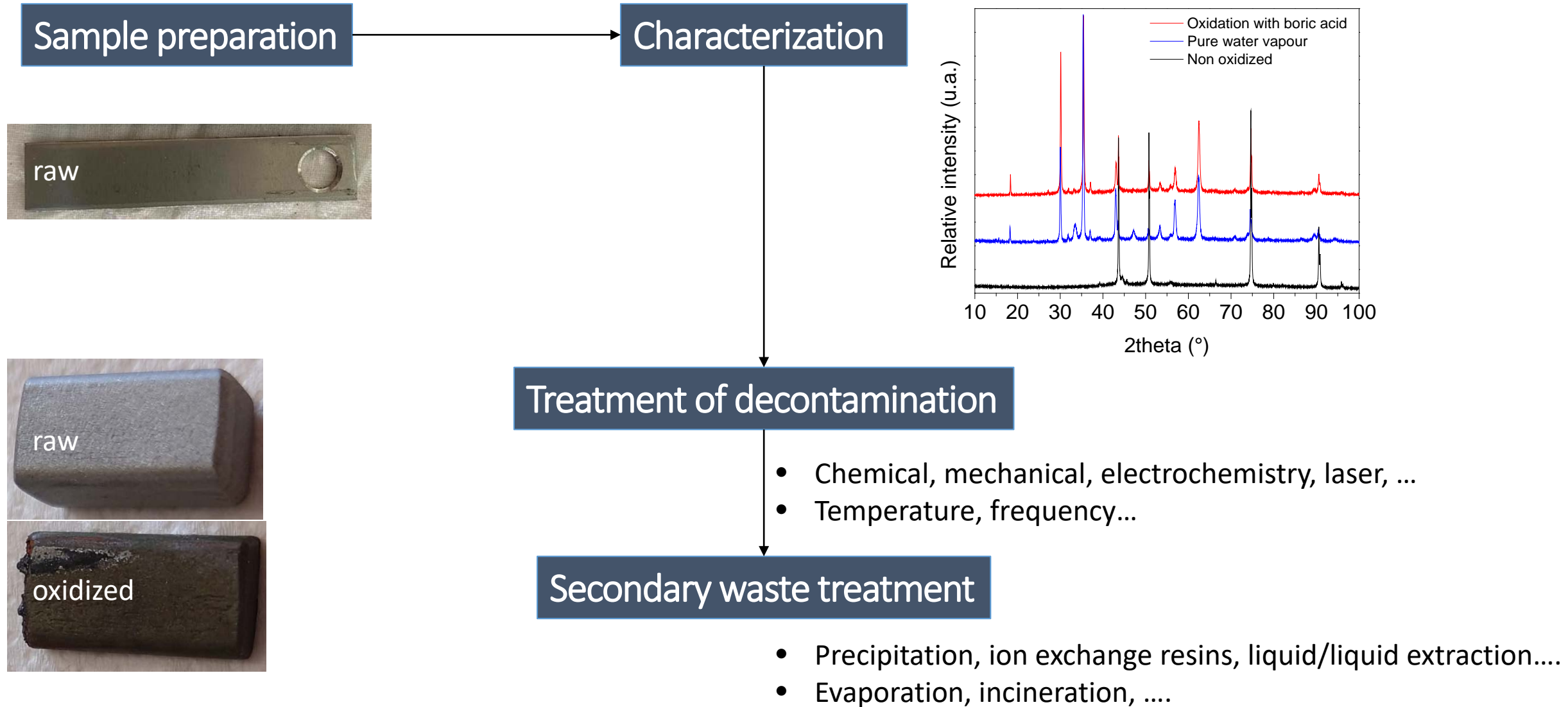


# Content



# Chemical metal decontamination

# Approach and Methodology



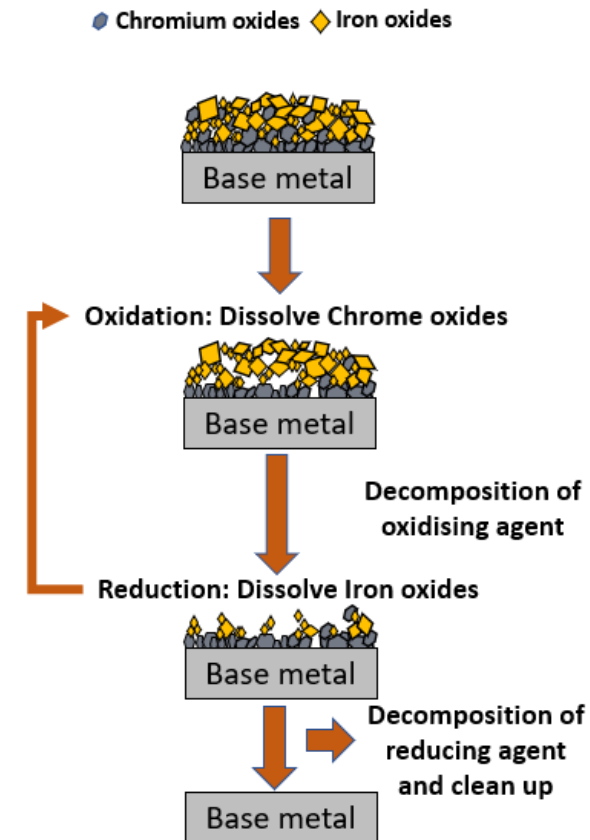
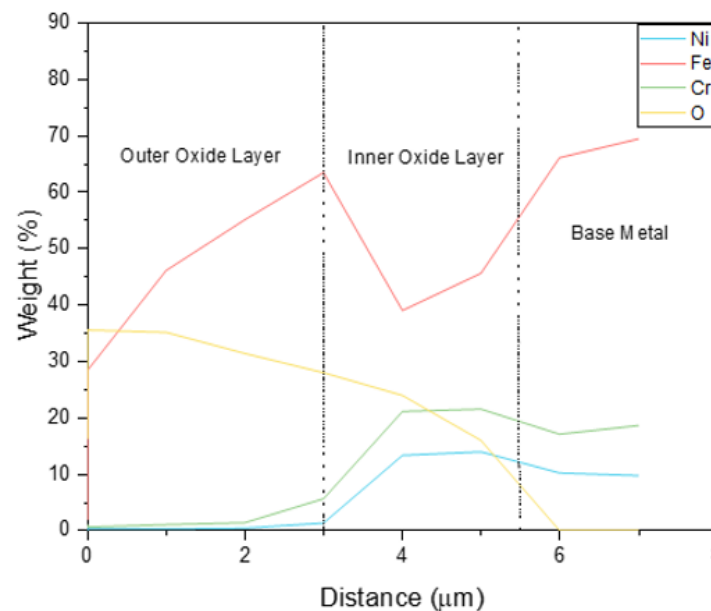
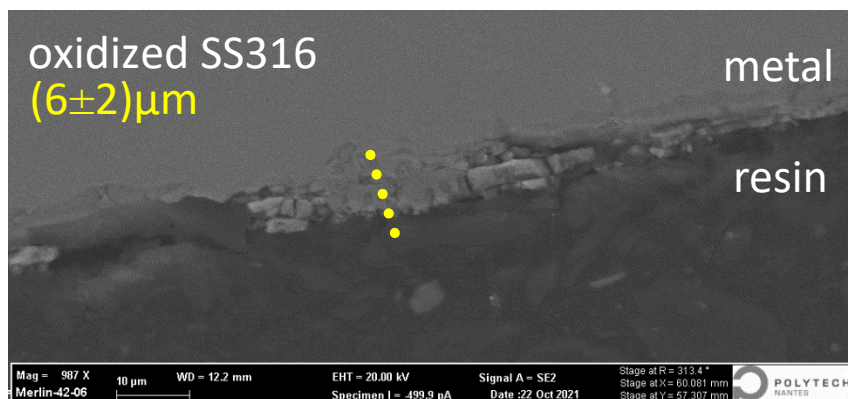
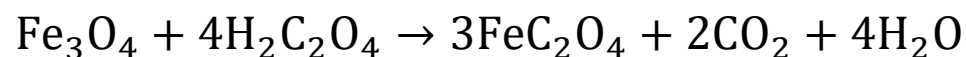
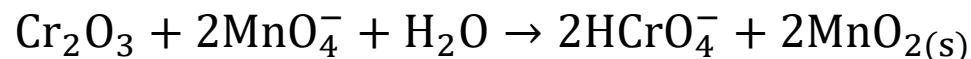
# Chemical decontamination

☐ **COREMIX process** : Chemical Oxidation Reduction Decontamination

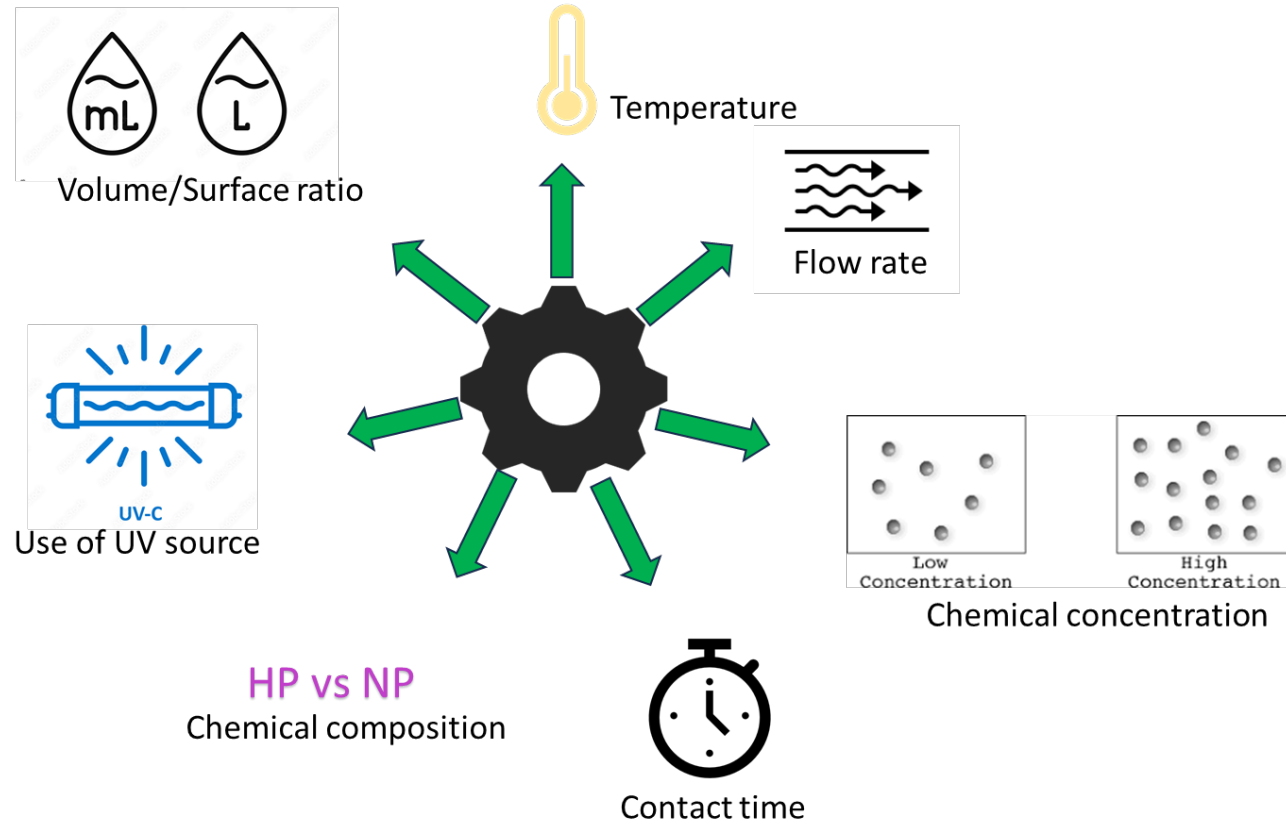
☐ Consecutive multi-step process:

1/ Pre-oxidation of the Cr oxide layer in the presence of permanganate

2/ oxidative dissolution of Fe oxides in the presence of oxalic acid



# Parameters evaluated



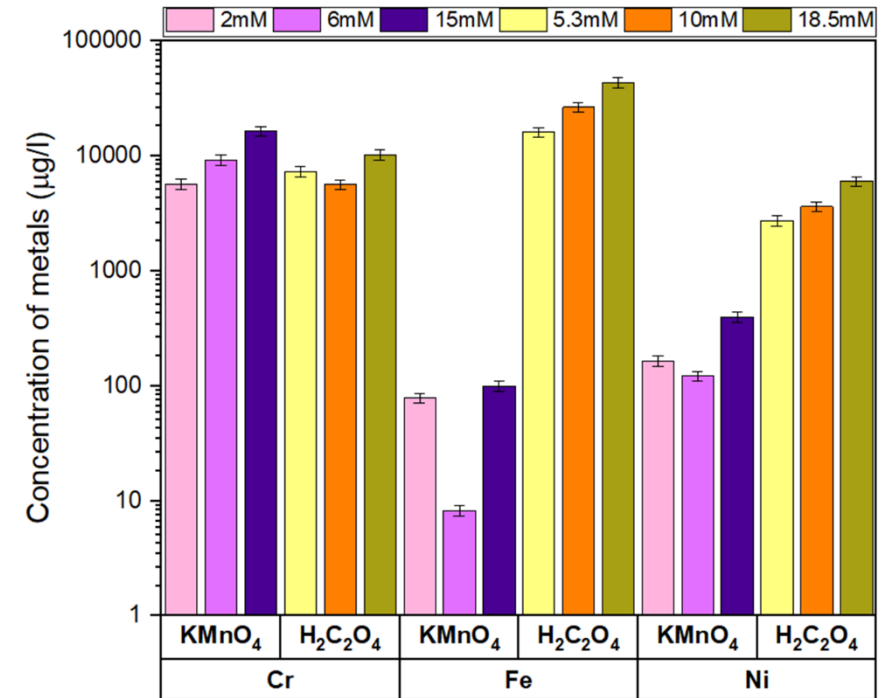
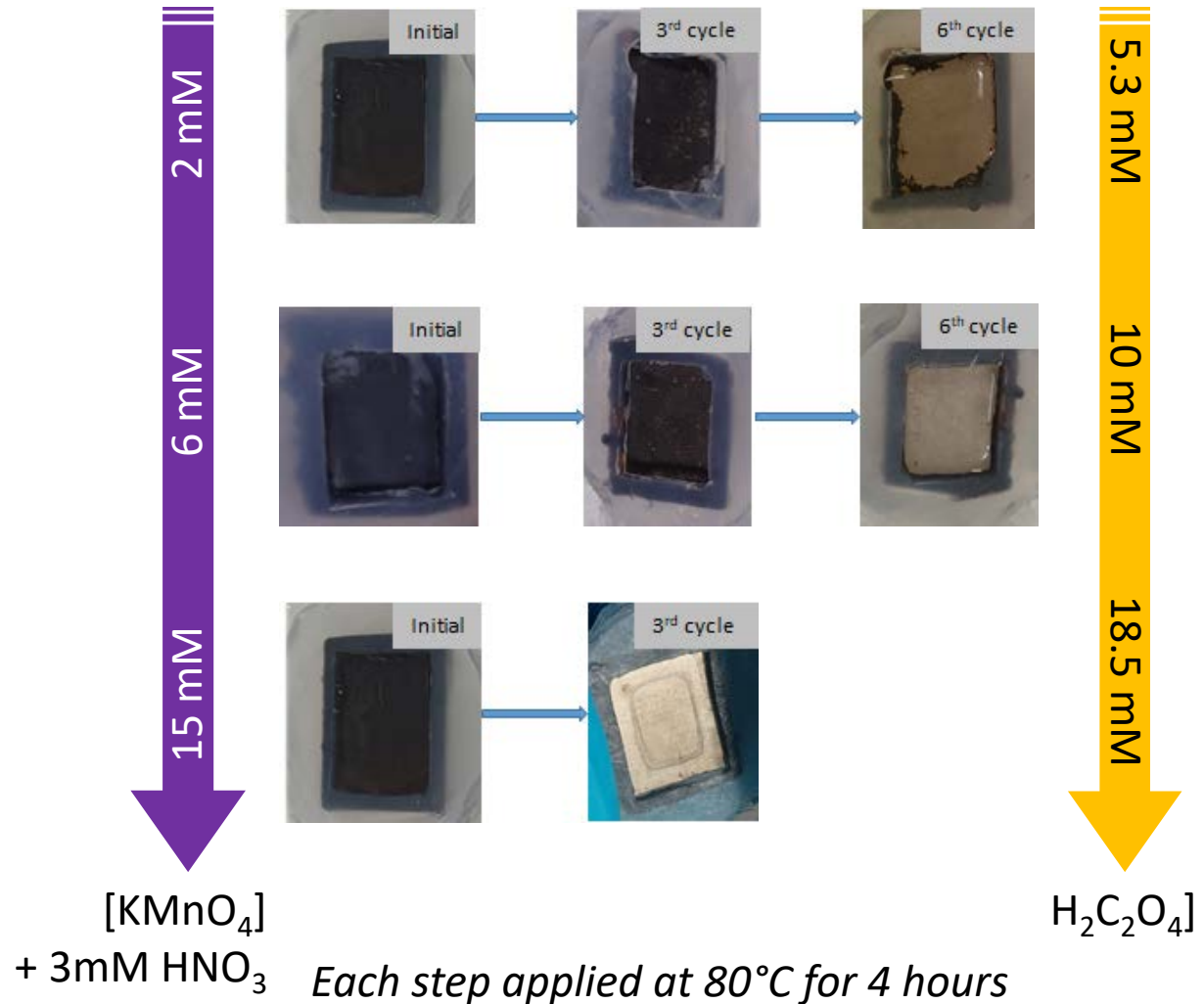
Non radioactive sample



Radioactive sample 

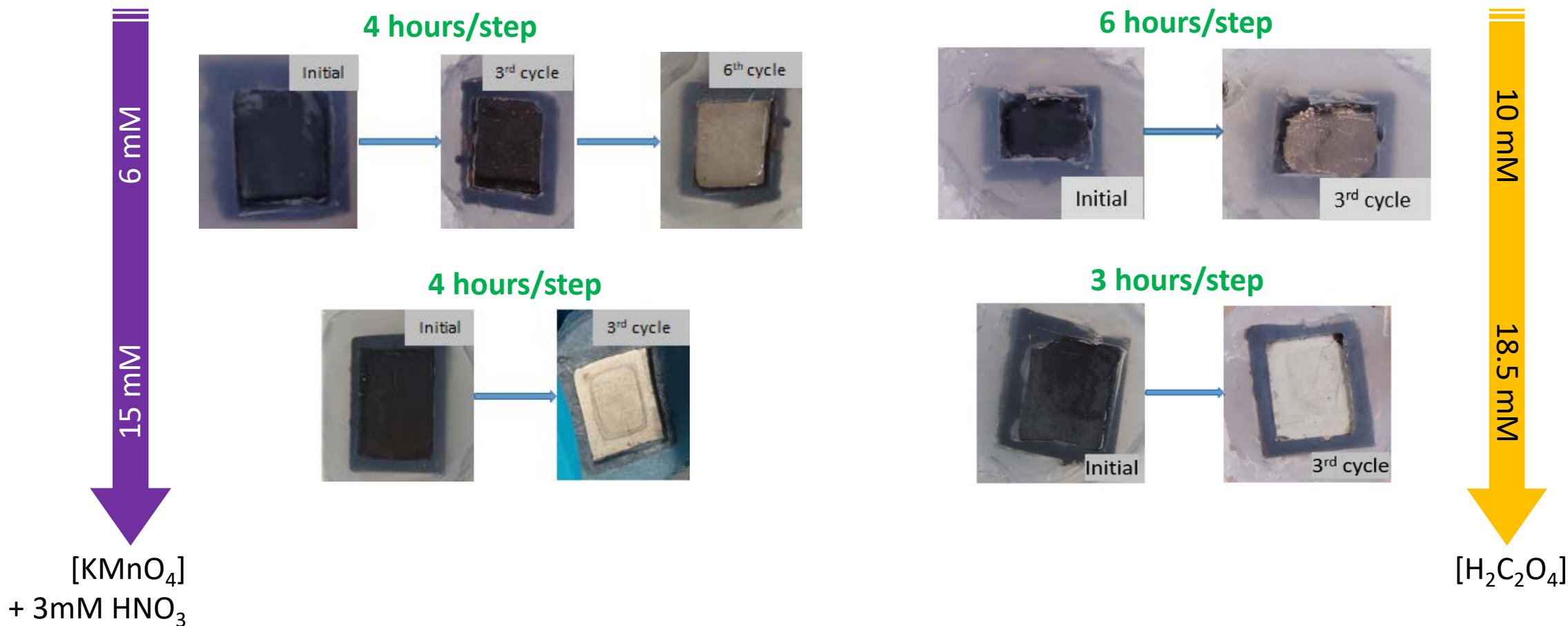


# Effect of chemical concentrations



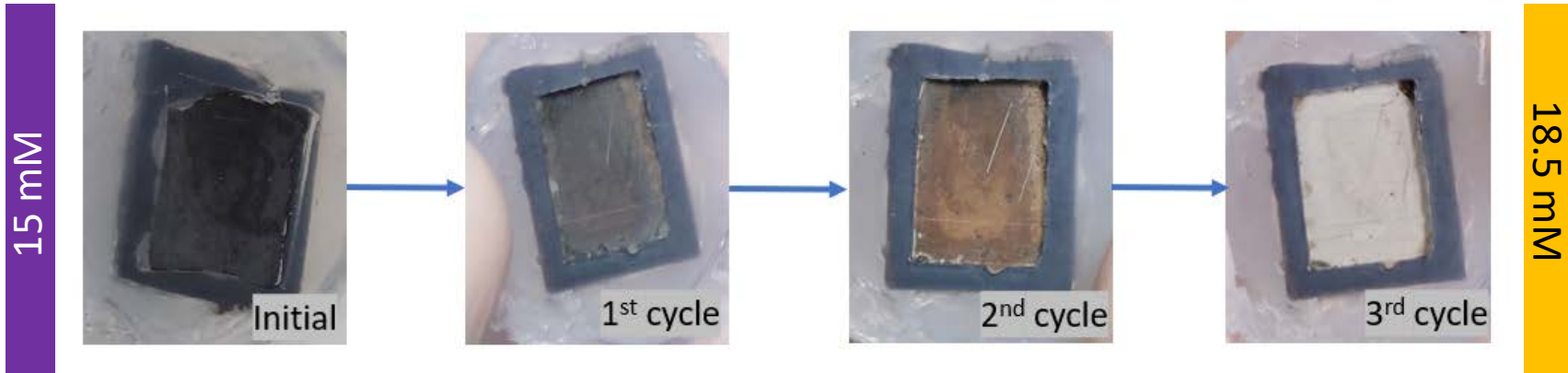
- Increasing concentration leads to reduction in number of cycles to 3

# Effect of contact time



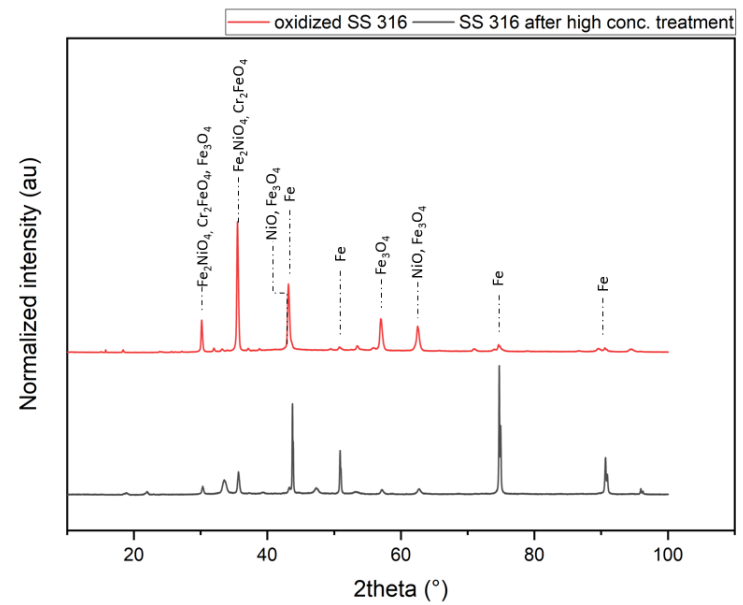
- Increasing concentration leads to reduction in contact time to 3 hours

# Evolution of sample surface

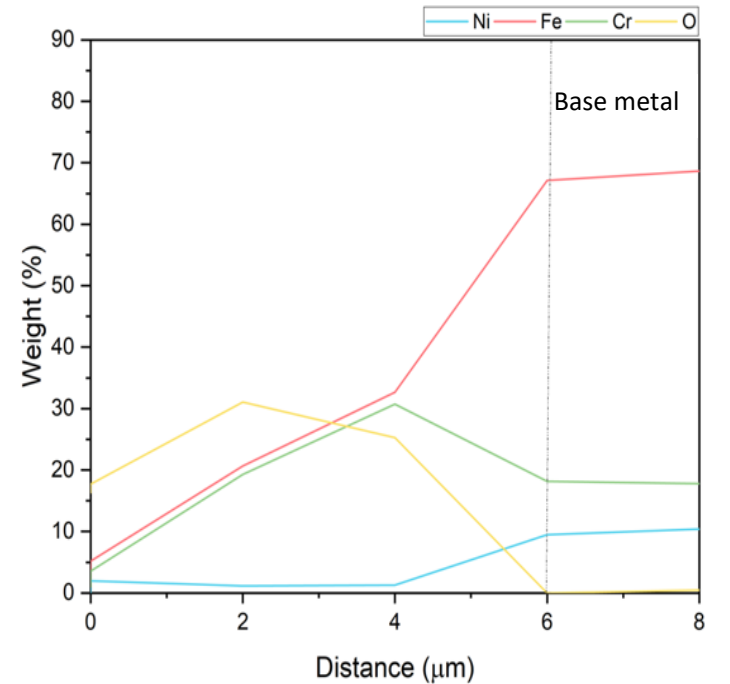
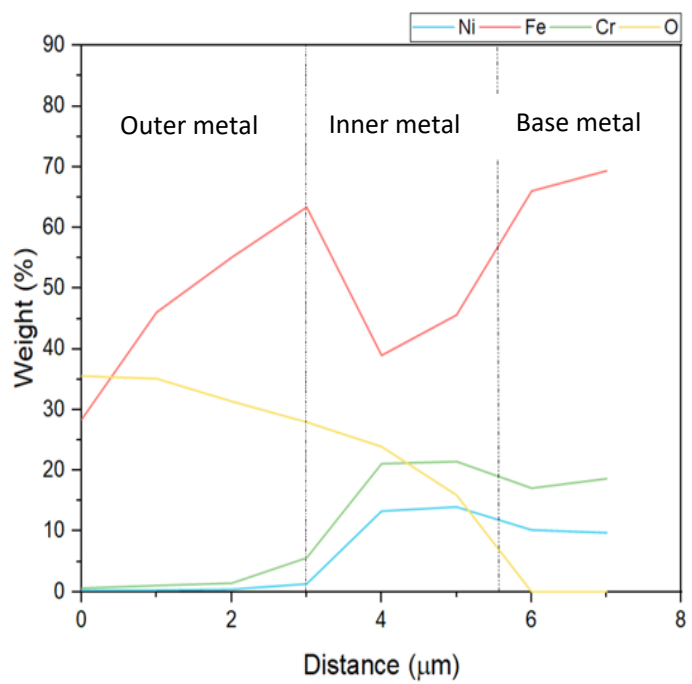


- Gradual dissolution of the oxide layer

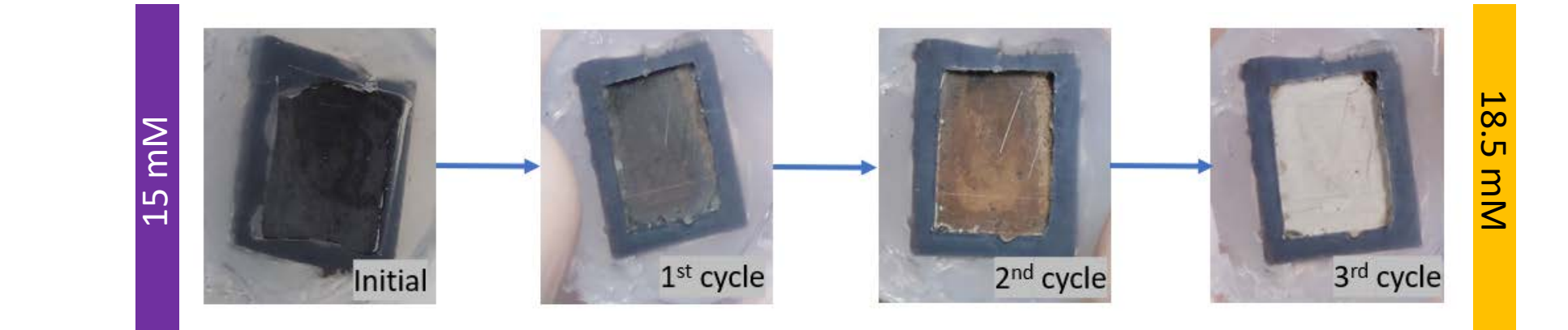
[KMnO<sub>4</sub>]  
 + 3mM HNO<sub>3</sub>



[H<sub>2</sub>C<sub>2</sub>O<sub>4</sub>]



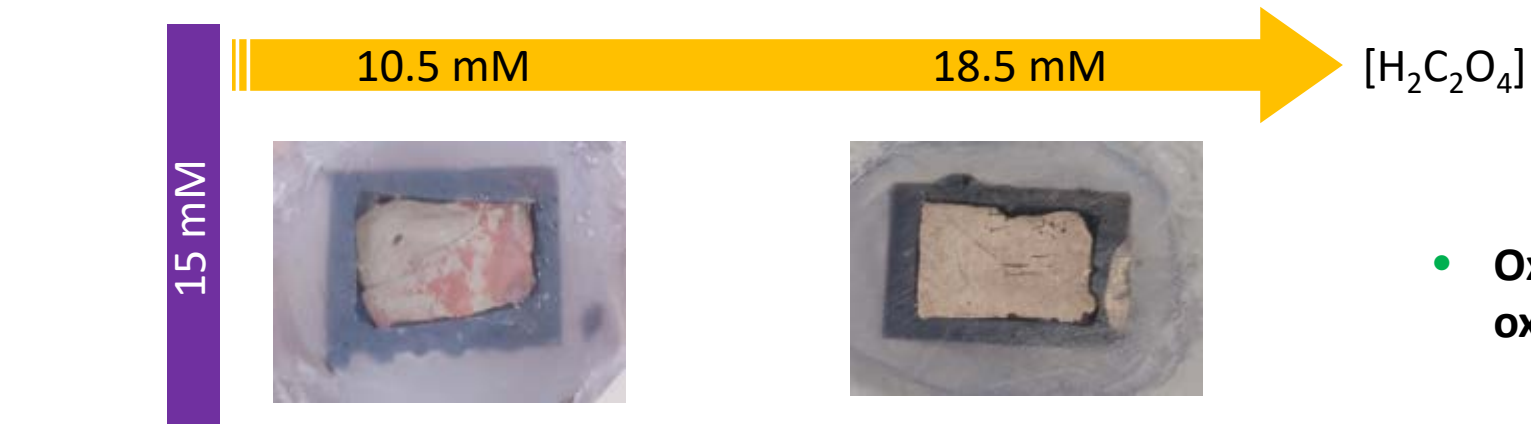
# Evolution of sample surface



[KMnO<sub>4</sub>]  
+ 3mM HNO<sub>3</sub>

[H<sub>2</sub>C<sub>2</sub>O<sub>4</sub>]

- Gradual dissolution of the oxide layer



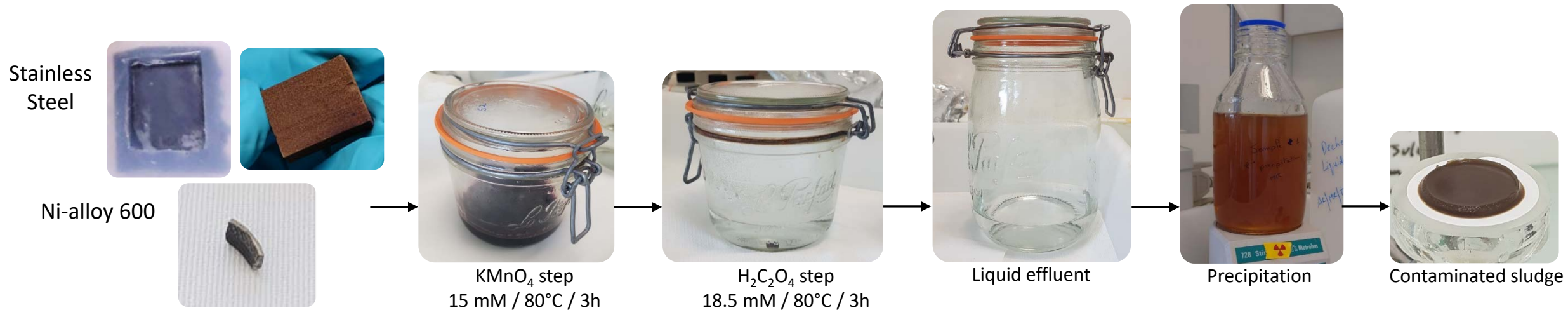
[KMnO<sub>4</sub>]  
+ 3mM HNO<sub>3</sub>

- Oxalic acid critical to oxide dissolution

# Treatment of secondary wastes

Liquid wastes from COREMIX process

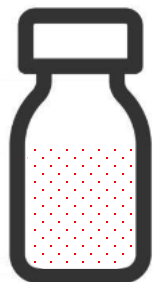
# From metal waste to liquid waste



- Reduction of secondary effluent volume by concentrating radionuclides in solid form



Radioactive effluent



+



Solid waste



Cationic resin

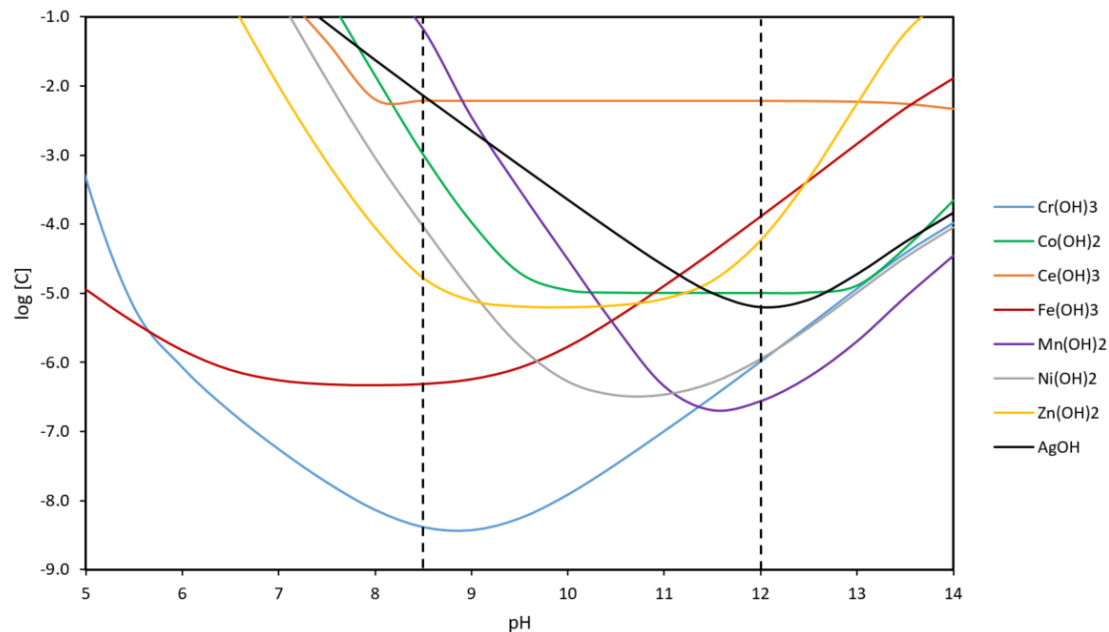
+



Decontaminated effluent

- Reduction in the amount of cationic resin required to remove residual radionuclides

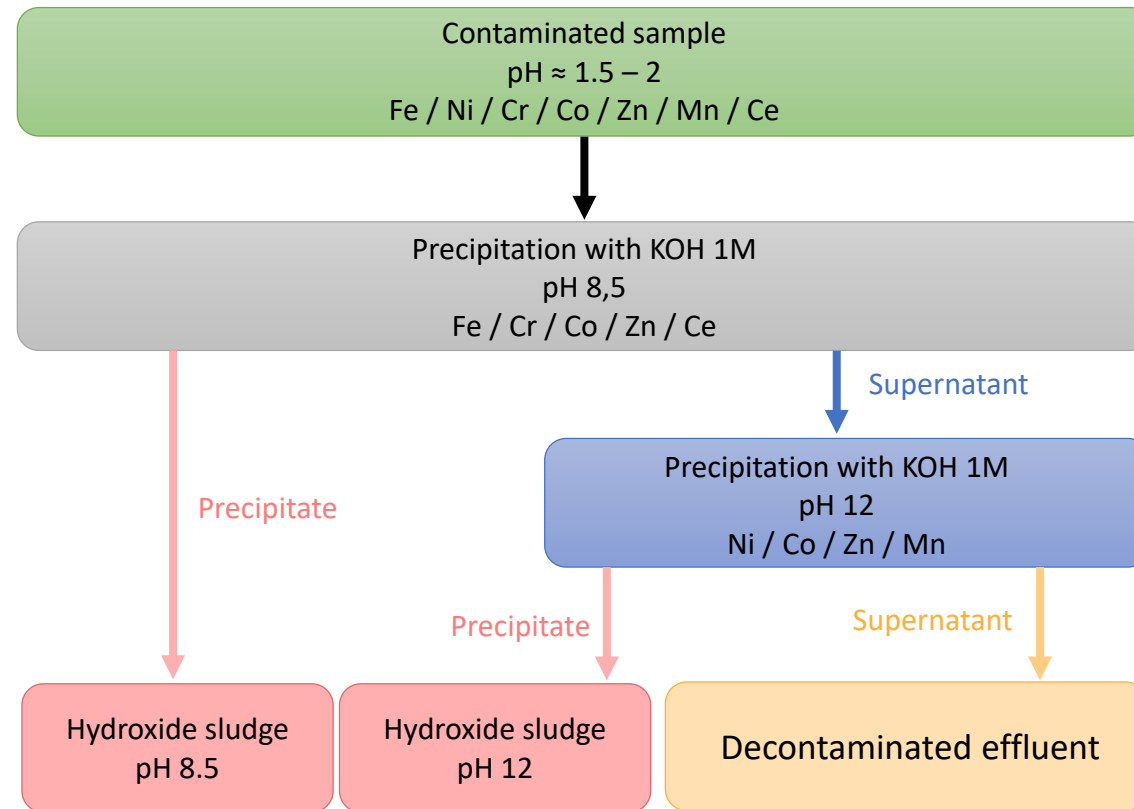
# Precipitation protocol



× Hydroxide precipitates are pH dependant

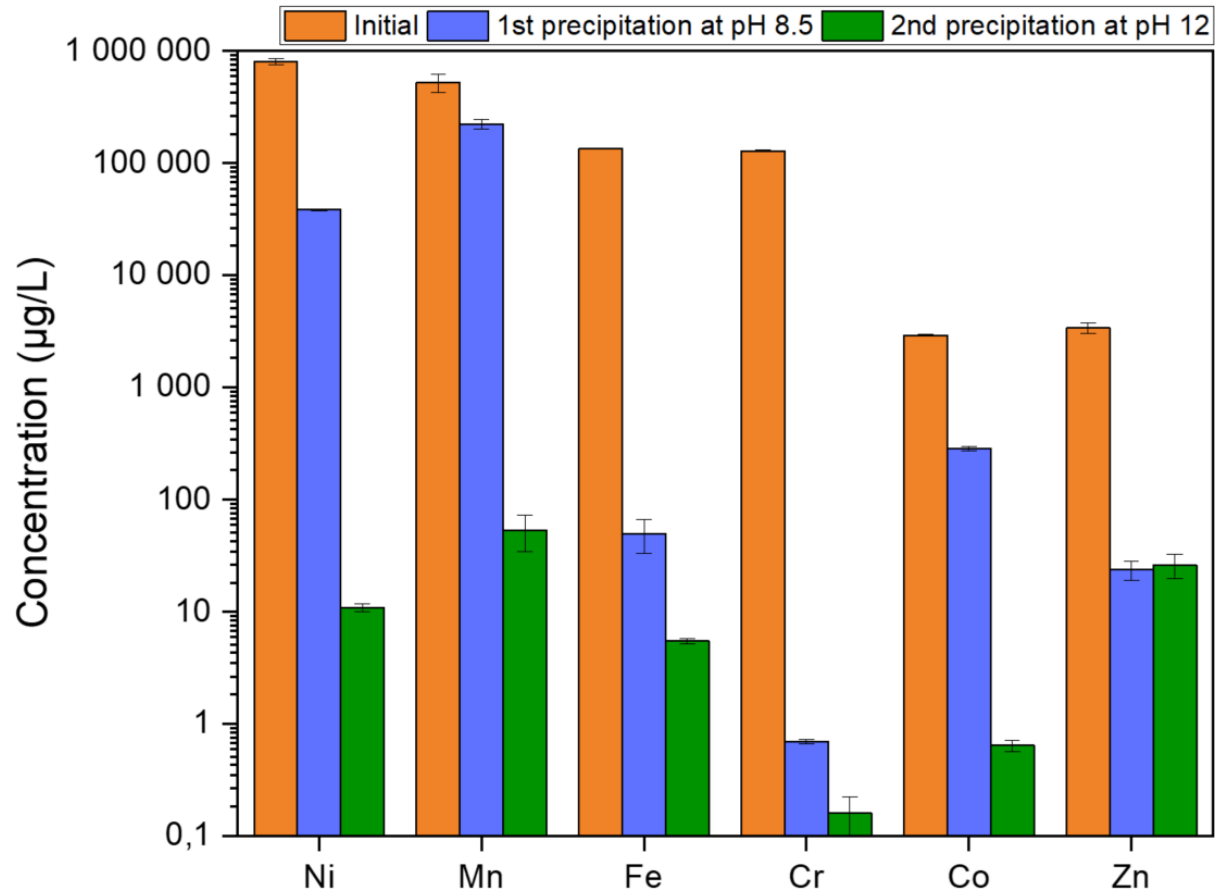
Several types of precipitates are possible :

- Oxide
- Hydroxide
- Sulphide
- Phosphate
- Chloride
- Fluoride



- ✓ Cheap reagent, fine for WAC and safe
- ✓ Environmentally friendly
- ✓ Adaptable at large scale

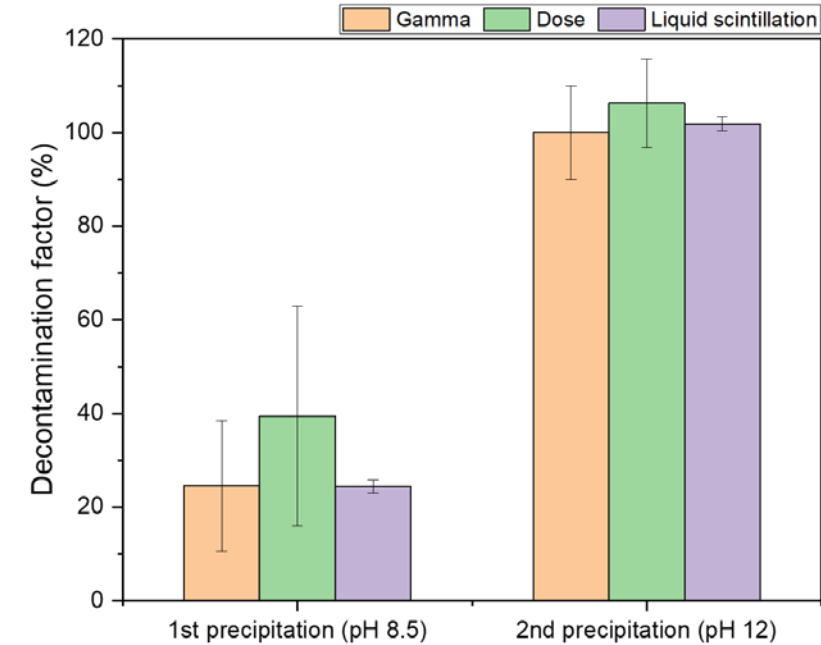
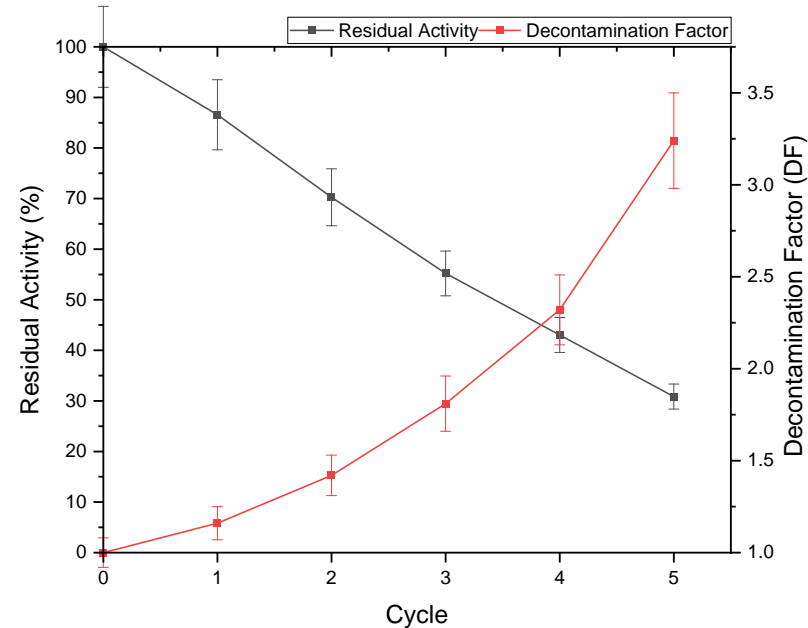
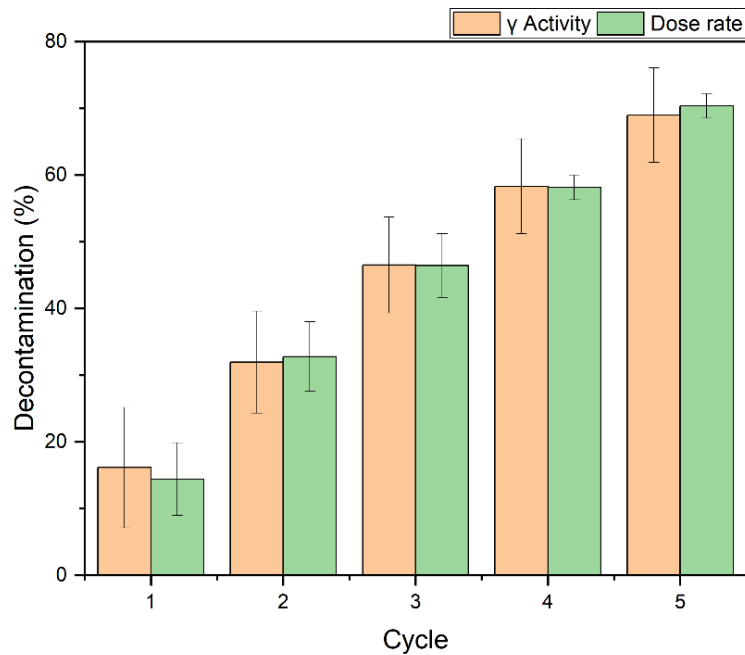
# Test on synthetic solution



<i>Elements</i>	<i>% precipitated pH 8.5</i>	<i>% precipitated pH 12</i>
$^{52}\text{Cr}$	100.0	100.0
$^{55}\text{Mn}$	48.4	100.0
$^{56}\text{Fe}$	100.0	100.0
$^{59}\text{Co}$	76.8	100.0
$^{60}\text{Ni}$	84.3	100.0
$^{66}\text{Zn}$	99.2	99.9
<b><i>Total efficiency</i></b>	<b>77.0</b>	<b>100.0</b>

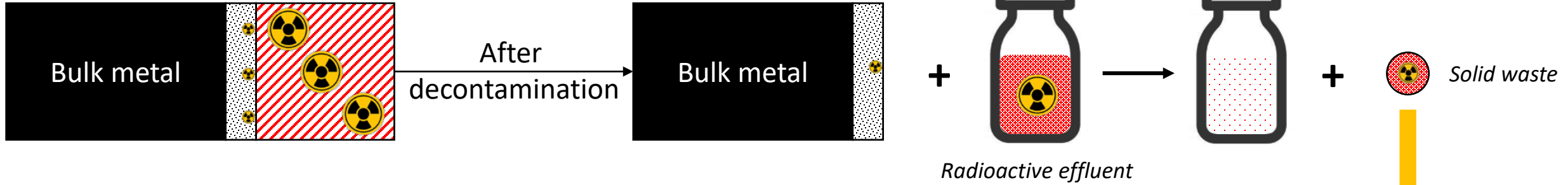


# COREMIX process applied on A600

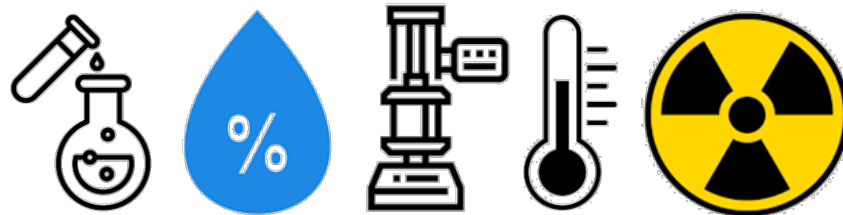
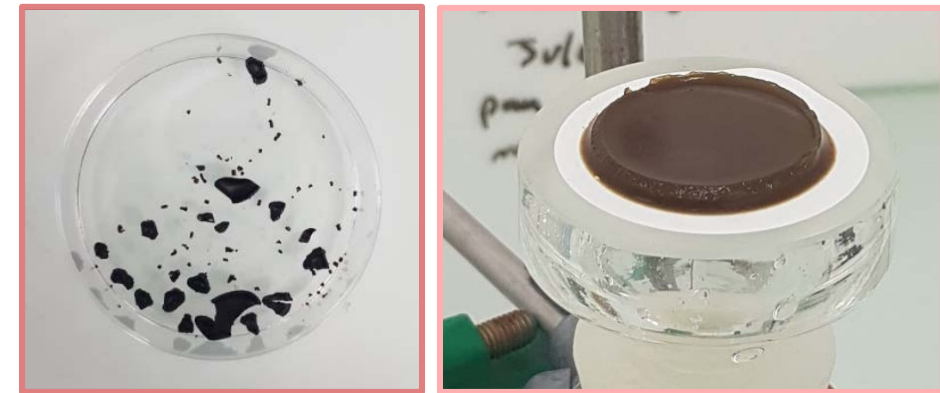


- **Metal decontamination**
  - 12%–14% decontamination per cycle  $\Rightarrow$  total of 8-9 cycles required for complete decontamination.
  - About 60%–70% decontamination achieved in 30 h of treatment
- **Liquid effluent decontamination**
  - pH 8.5 :  $\sim$ 24% drop in total radioactivity  $\Leftarrow$  Mainly coming from  $^{55}\text{Fe}$  removal
  - pH 12:  $\sim$ 100% of decontamination  $\Leftarrow$  Removal of  $^{60}\text{Co}$  and  $^{63}\text{Ni}$

# Conditioning secondary waste



- **Inovative matrixes:** geopolymers, magnesium cement phosphate, ...



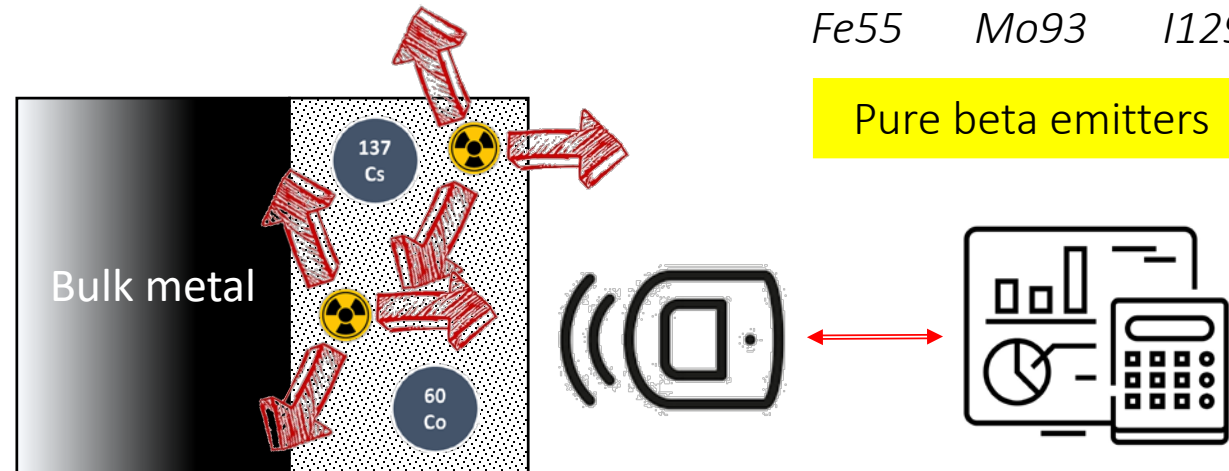
# Difficult to measure radionuclides

DTM

# What is a DTM?



- ❑ DTMs characterized by their low energy emission, and/or low intensity of emission
- ❑ Evaluation of the activities by scaling factors to determine the activity level
  - Non destructive approach
- ❑ Difficulty to measure directly pure beta emitters :
  - destructive method generally used.
- ❑ Need of destructive approach to:
  - Direct measurement of radioactivity
  - Validation of scaling factor models
  - Minimization of uncertainties



<i>Be10</i>	<i>Zr93</i>	<i>Ni59</i>
<i>C14</i>	<i>Nb94</i>	<i>Ni63</i>
<i>Cl36</i>	<i>Tc99</i>	<i>Se79</i>
<i>Ca41</i>	<i>Pd107</i>	<i>Sr90</i>
<i>Fe55</i>	<i>Mo93</i>	<i>I129</i>

Pure beta emitters

- ⇒ Development of a **radiochemical procedure** for the optimal detection and measurement in metallic sample
- ⇒ **Highly selective** and efficient separation and purification
- ⇒ Development of **sensitive** to **ultra-sensitive** method of measurement, depending on the radionuclide

# Example: zirconium-93

## ☐ a long-lived RN ( $1.61 \times 10^6$ years)

- Fission product of uranium and plutonium.
- Neutron activation product of stable Zr in PWR nuclear fuel cladding.
- Contribution to the total waste inventory: dominates after 1000 years with Tc-99.

## ☐ decays to stable Nb-93 by beta emission

- but with a low energy :  $E_{\max}$  60Kev
- $E_{\max}$  : 60.0 keV (73%) et 90.8 keV (27%)
- Analyses by
  - LSC: require a very good chemical separation prior the measurement
  - ICP-MS: possible presence of isobaric interfering elements (Mo-93 and Nb-93)

## ☐ element to monitor

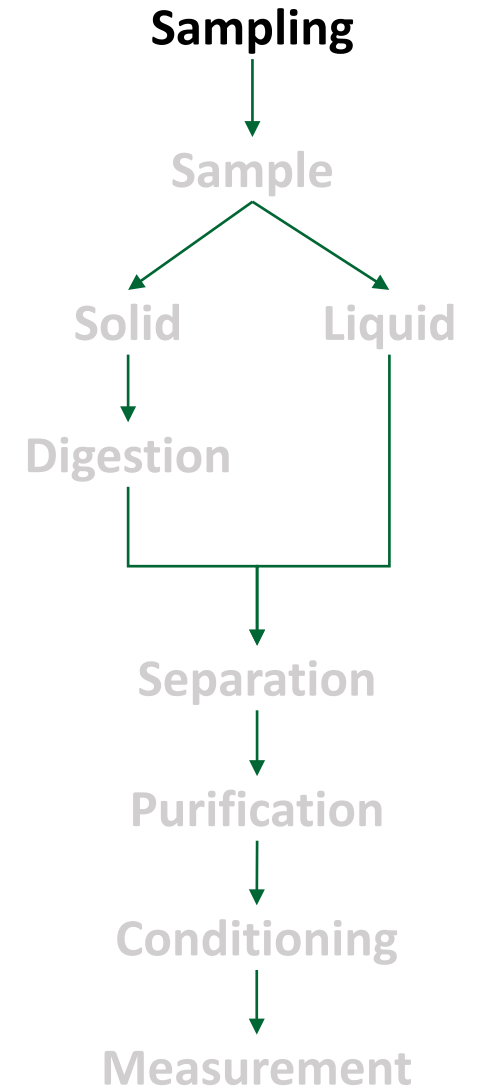
- Decommissioning and characterization of nuclear sites
- Release to the environment

⇒ no standard procedure, no standard source

# General methodology & Challenges

## □ Sampling

- Representative samples
  - mass/volume
  - high/low activity
  - heterogeneity/homogeneity
  - Location, access
- Selection of sampling technique
  - Drill, saw, cutter etc.
  - Geometry/form of the sample
- Cross contamination
  - What will be studied?
  - Contamination from the sampling technique?

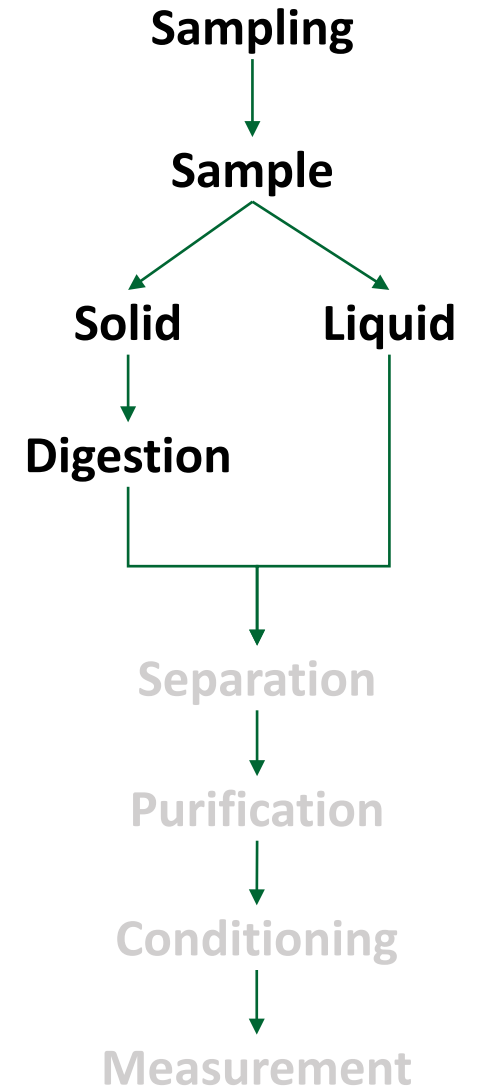
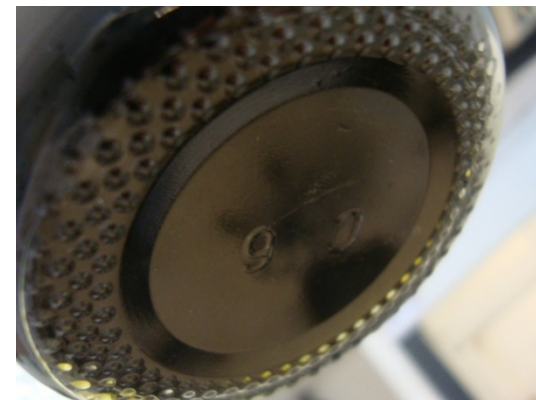
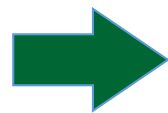


# General methodology & Challenges

☐ Sampling

☐ Sample preparation

- DTM extraction from metal: acid digestion, thermal oxidation, ....
- Radionuclide specific restrictions: volatility, activity level
- Chemical constrains: solubility, chemical composition....
- Total digestion?
- Synthetic solution, surrogate sample





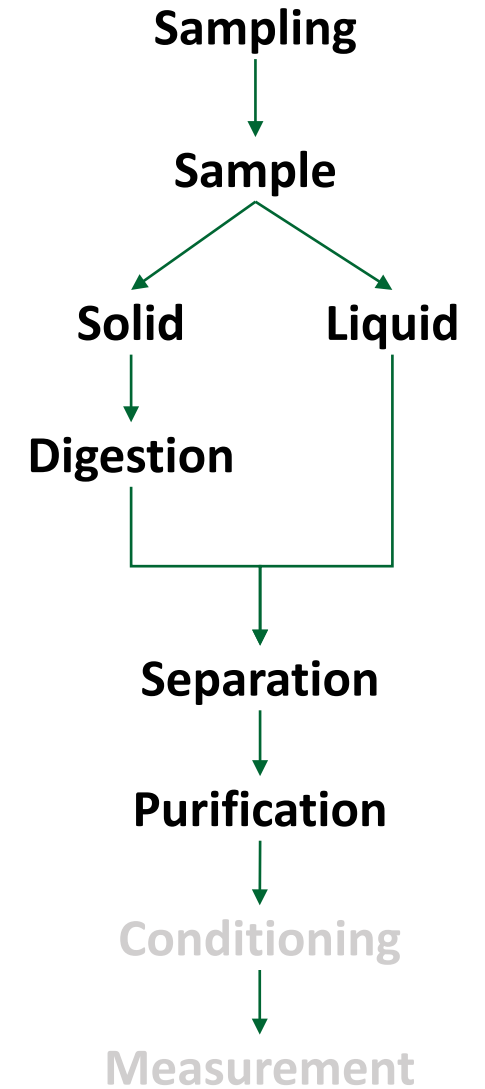
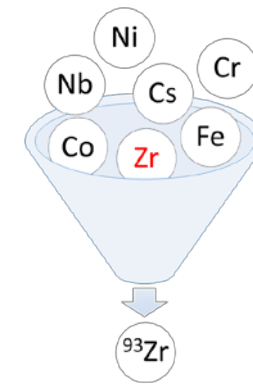
# General methodology & Challenges

☐ Sampling

☐ Sample preparation

☐ Separation – Purification

- Chromatographic resins
- Liquid/liquid extraction
- Consideration of interferences during purification process and detection





# General methodology & Challenges

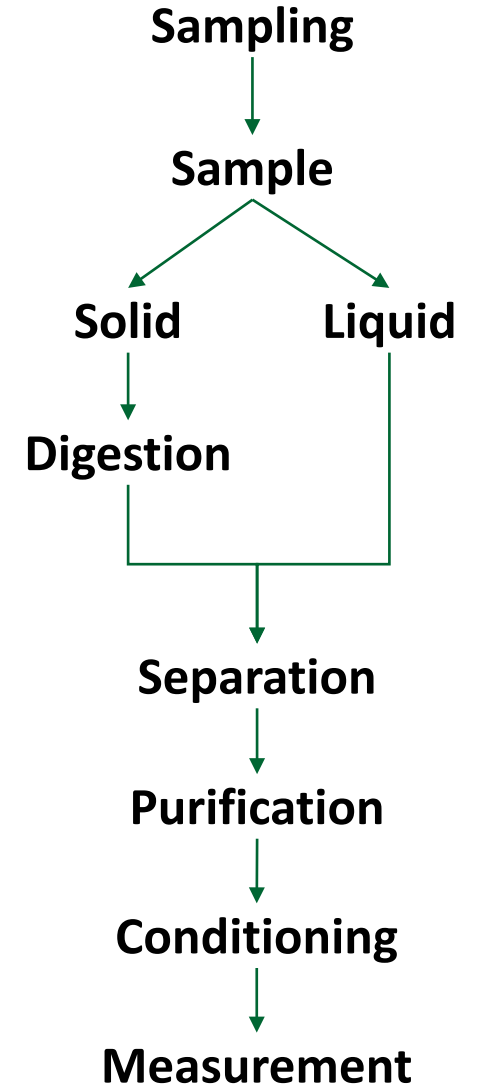
Sampling

Sample preparation

Separation – Purification

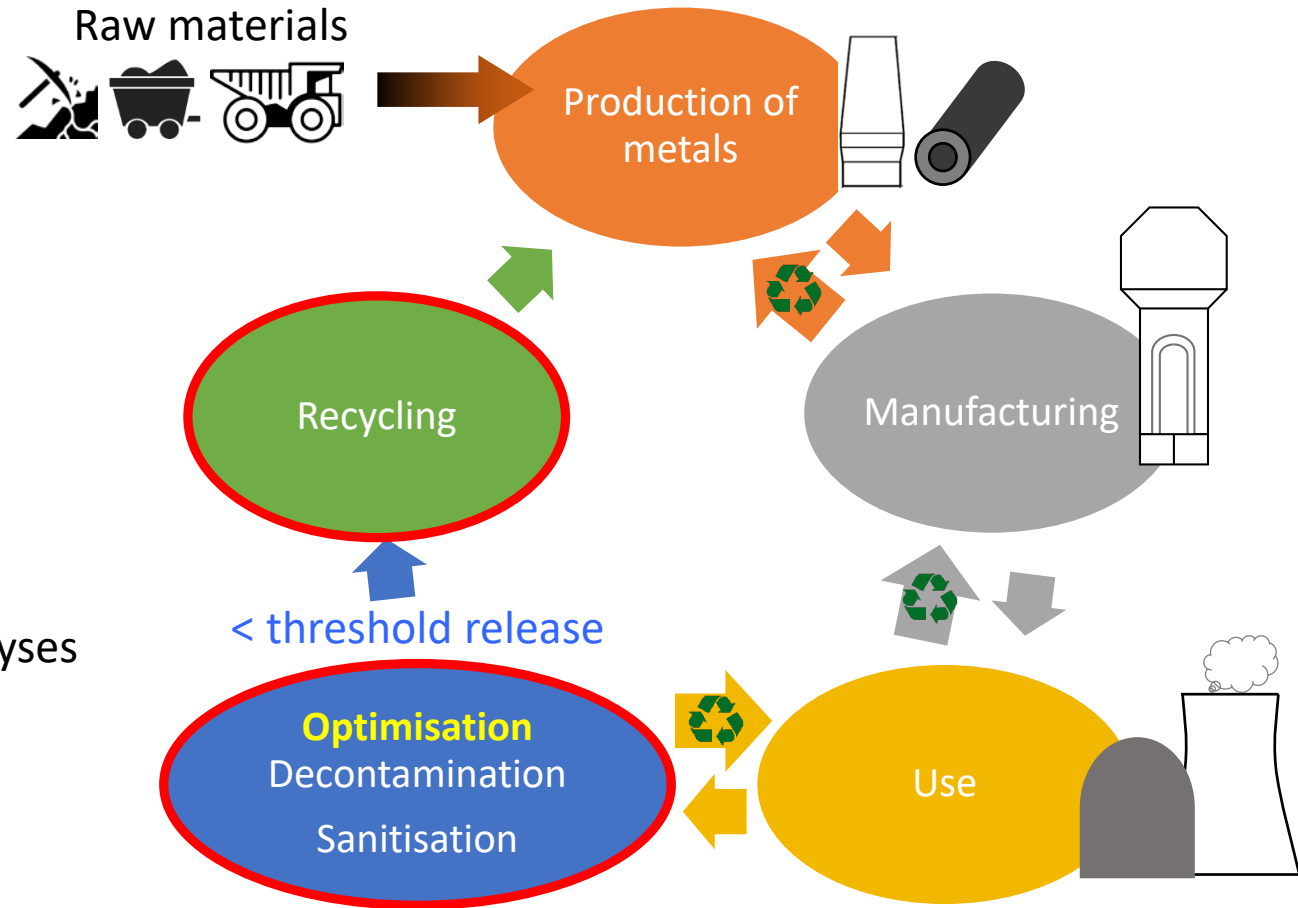
Sample conditioning and measurement

- Liquid form or solid form
- Selection of an adapted analytical technique for measurements
  - Liquid scintillation counting: quenching, contamination
  - X-ray: BEGe gamma spectrometer,
  - Mass spectrometry: ICP, AMS (limited), ...
  - Lowering detection limits, optimization of detection efficiency



# To summarize

- Reduction of volume waste
- Saving raw materials
- Developing innovative matrixes
- Developing new procedures for DTM analyses



# Acknowledgements

## ☐ [Subatech laboratory](#)

- Radiochemistry team
- SMART team

## ☐ WP4 partners of [PREDIS Project](#)

- VTT, NNL, SORC, CEA, UJV, CTU, POLIMI

## ☐ [Triskem company](#)