

# Technical Workshop on Fuel Cycle Simulation

## Functionality Isolation Test (FIT)

09 / 07 / 2018

N. Thiollière<sup>1</sup>,  
B. Mouginot<sup>2</sup>,  
X. Doligez<sup>3</sup>,  
B. Feng<sup>4</sup>,  
E. Davidson<sup>5</sup>,  
J. Whan Bae<sup>5</sup>,  
R. Hays<sup>6</sup>,

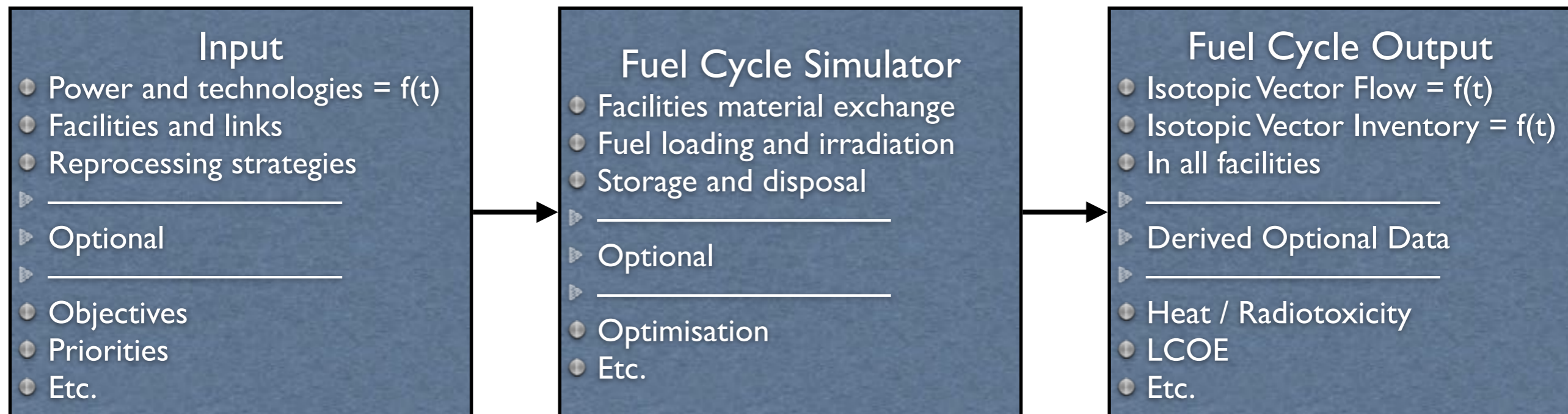


- 1 - Subatech, IMTA-IN2P3/CNRS-Université, Nantes, France
- 2 - University of Wisconsin-Madison, USA
- 3 - IPNO, CNRS-IN2P3/Univ. Paris Sud, France
- 4 - Argonne National Laboratory, U.S.A.
- 5 - Oak Ridge National Laboratory, U.S.A.
- 6 - Idaho National Laboratory, U.S.A.



# Fuel Cycle Simulators

- Since the 90's, a lot of different fuel cycle tools have been developed
  - Institutions : Industrial, Engineering, Academic
  - Complexity : From the simple spread sheet to the complex simulation framework
  - Capabilities and Flexibility : One specific problem to any problems



- Dynamic fuel cycle simulators are used for several applications :
  - Part of the technical evaluation of innovative systems deployment
  - Identification of drivers / parameters interactions in fuel cycle fleet physics
  - Production of data for further assessments (economy, safety, non-proliferation, etc.)

# Improving the confidence in fuel cycle simulators output

## Uncertainty assessment / propagation

### Uncertainties

- G. Krivtchik PhD : Nuclear data impact
- A. Somaini PhD : Systems simplifications
- Scenario simplifications

### Operational data

- F. Courtin PhD : Global Sensitivity Analysis

### Scenario usage

- Problem formulation
- Problem resolution / methodology

## Link between scenario and decision

### Sociology : Role of scenarios

- Interviews with users
- Focus Groups with actors
- Round Table with decision maker

## Comparison with experimental data

### Reactor Data

- Possible to validate PWR UOx models
- Complex to assess PWR MOx models
- No available data for innovating reactors

### Fleet Data

- Complex history with lack on input data
- Lack of output data

## Code testing / comparison

### Code comparison

- NEA benchmarks (2012)
- MIT Benchmark (2009)
- IAEA/INPRO Programme (2013)

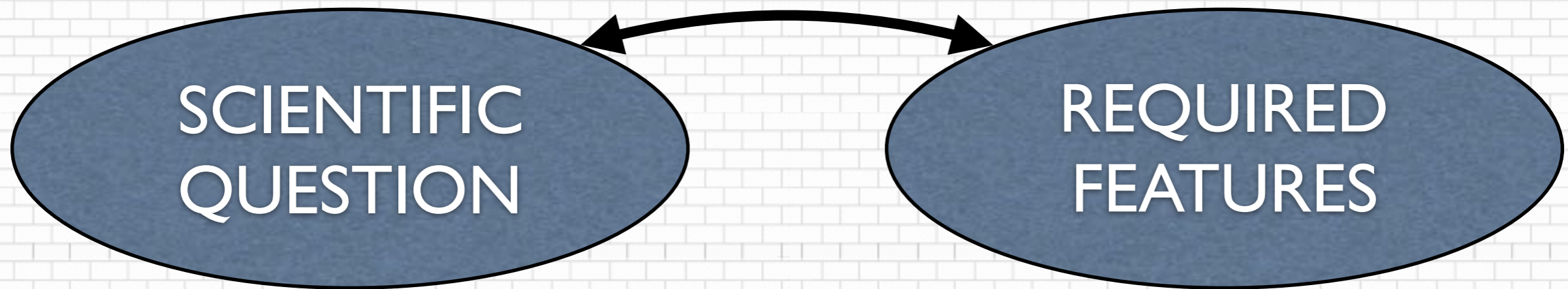
### Code testing methods

- Unit tests

### Functionalities impact

- FIT Project

# FIT (Functionality Isolation Test) Project description



- The FIT effort is another brick in the wall of fuel cycle output confidence:
  - Focus on one single fuel cycle simulator functionality
  - Based on **KISS** (Keep It Simple Scientist!) principle

- Recipe
- Continuous reprocessing
- No decay
- Agent based reactors
- A single composition
- Average load factor
- Constant fuel worth

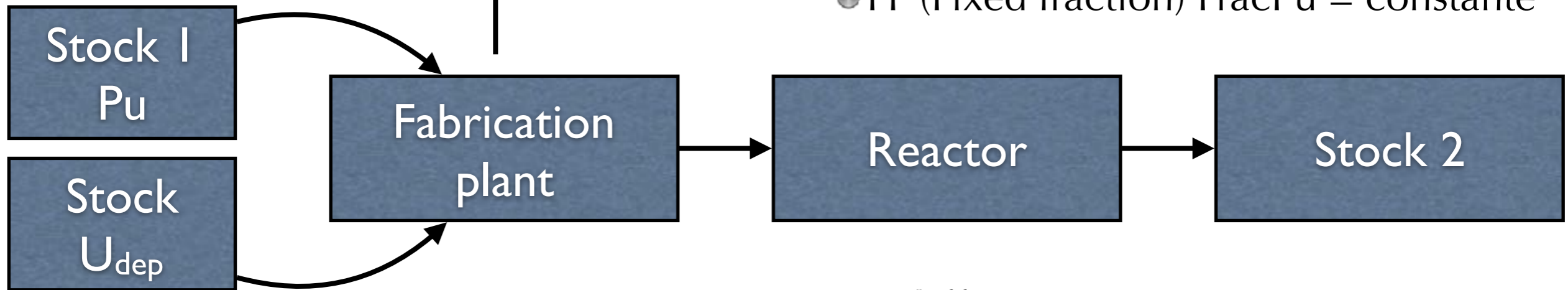
VERSUS

- Update fuel composition
- Reprocessing when needed
- Isotopic decay
- Fleet based reactors
- Exact start-up composition
- Load following
- Physics of mixed core



# Testing Update fuel composition VS recipe

Exercice design



2 strategies for building the fuel :

- FLM (Fuel Loading Model)  $\text{FracPu} = f(\text{BU})$
- FF (Fixed fraction)  $\text{FracPu} = \text{constante}$

● 4 different compositions

- Reference is tuned to get FLM = FF
- Three test compositions

Parameters	SFR	PWR
Power	2500 MW <sub>th</sub>	3000 MW <sub>th</sub>
Capacity factor	0.9	0.9
Cycle length	1.264 y	1 y
Number of batch	5	3
Fuel residence time	6.32 y	3 y
Core HM mass	51953.4 kg	72000 kg
Annual HM loading	8218.1 kg	24000 kg

Weight %	Reference	Compo 1	Compo 2	Compo 3
Pu-238	1.98	3.12	2.87	4
Pu-239	62.25	51.59	46.99	38.53
Pu-240	22.50	24.32	33.91	24.56
Pu-241	8.00	11.75	4.54	15.9
Pu-242	5.00	8.04	10.92	12.78
Am-241	0.27	1.18	0.77	4.23

# Results For The Reactor Loading

## ● FLM Results - PWR

CLASS - Neural Network				
Frac Pu	7	8.7	15.3	12
K <sub>threshold</sub>	1.03	1.03	1.03	1.03
CYCLUS - Pu equivalent				
Frac Pu	7	7.9	10.9	9.4

## ● FLM Results - SFR

CLASS - Neural Network				
Frac Pu	16	14.5	18.1	14.7
K <sub>threshold</sub>	1.03	1.03	1.03	1.03
CYCLUS - Pu equivalent				
Frac Pu	16	15.6	22.6	16.4

## ● FF impact on K<sub>threshold</sub>

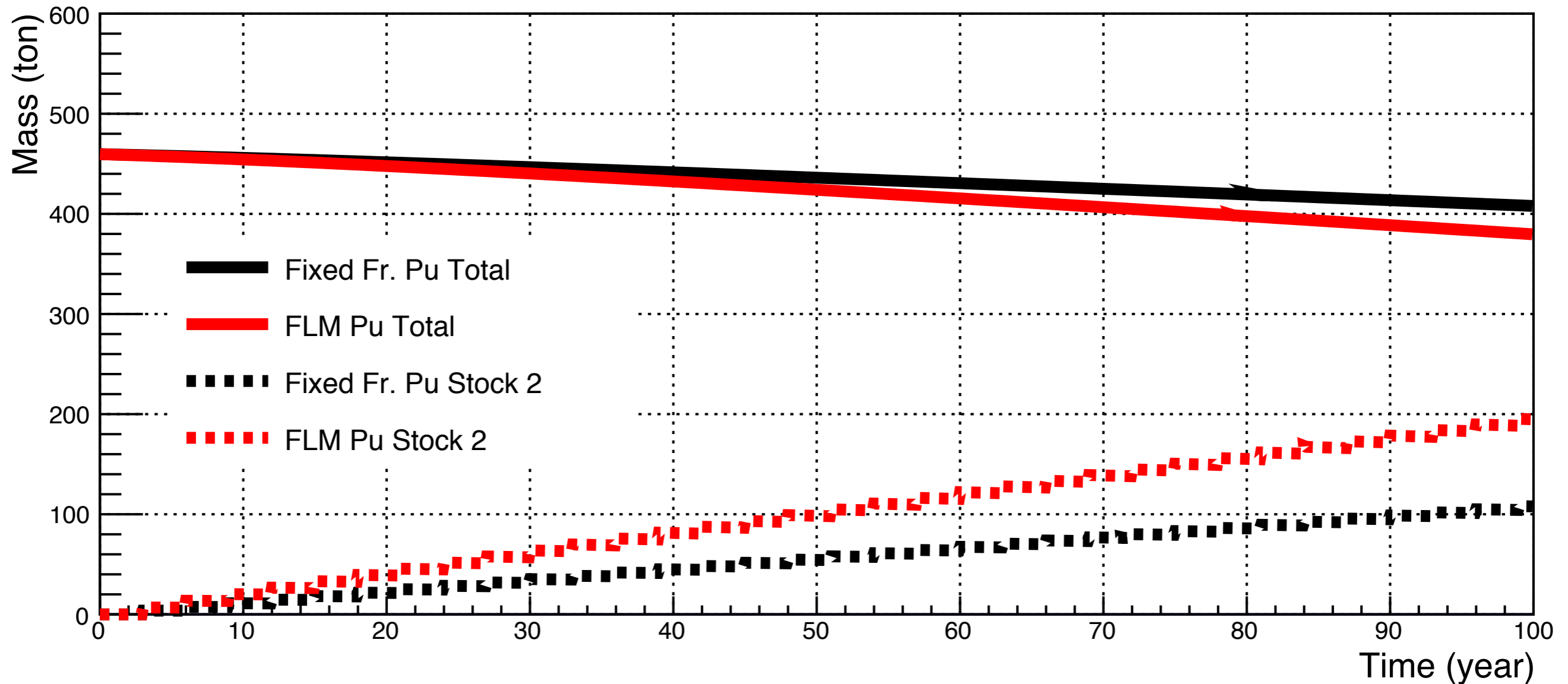
CLASS - Neural Network				
Frac Pu	7	7	7	7
K <sub>threshold</sub>	1.03	1.005	0.959	0.97

## ● FF impact on K<sub>threshold</sub>

CLASS - Neural Network				
Frac Pu	16	16	16	16
K <sub>threshold</sub>	1.032	1.09	0.96	1.08

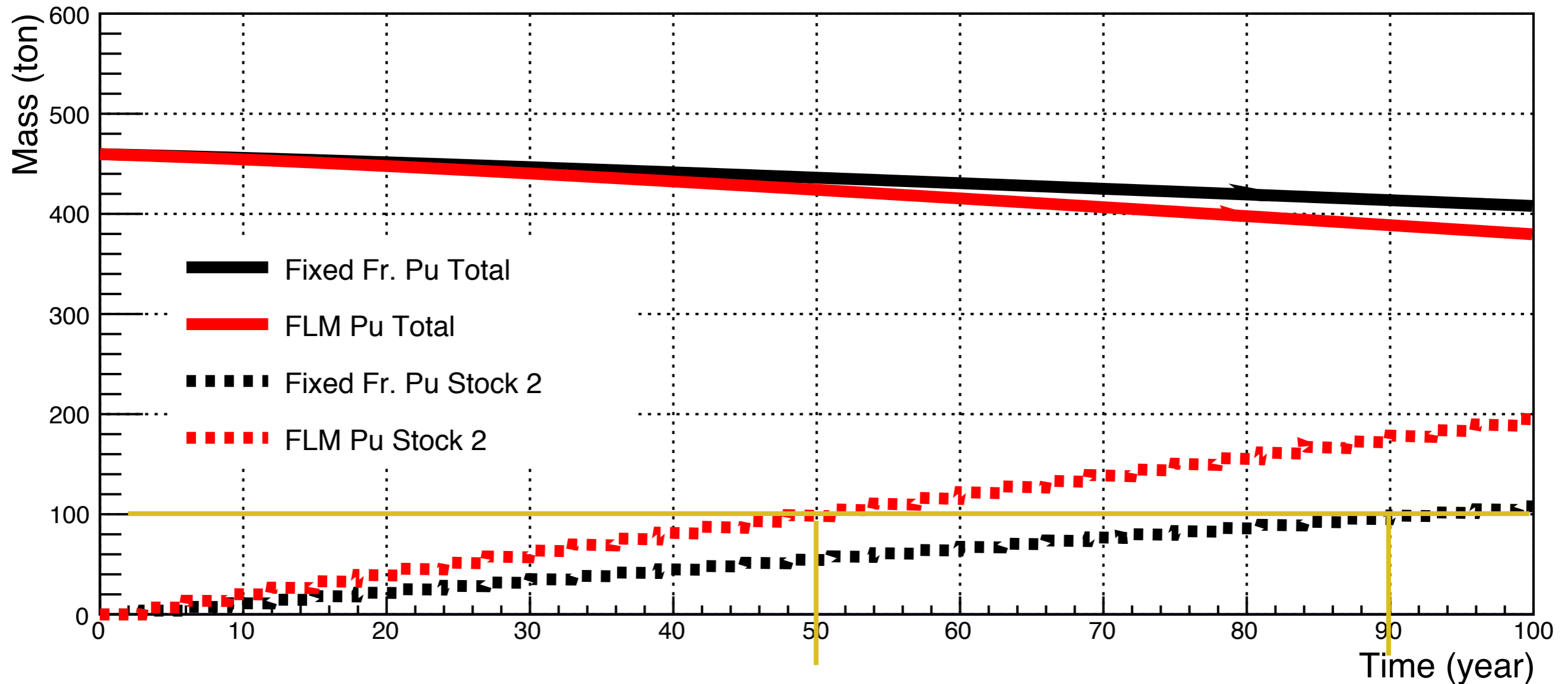
- What is the impact of reactor deviation on a fuel cycle calculation?
  - ▶ One Reactor for 100 years operation
  - ▶ No decay in the stock 1: Same Fuel @ BOC

# Results for PWR - MOx Compo 3 / CLASS



- The deviation between Total Plutonium FF and FLM is acceptable
  - ▶ Burn-up are similar so Pu balance are comparable
- The deviation between Stock 2 Plutonium FF and FLM is important
  - ▶ Pu fraction at B.O.C. are very different

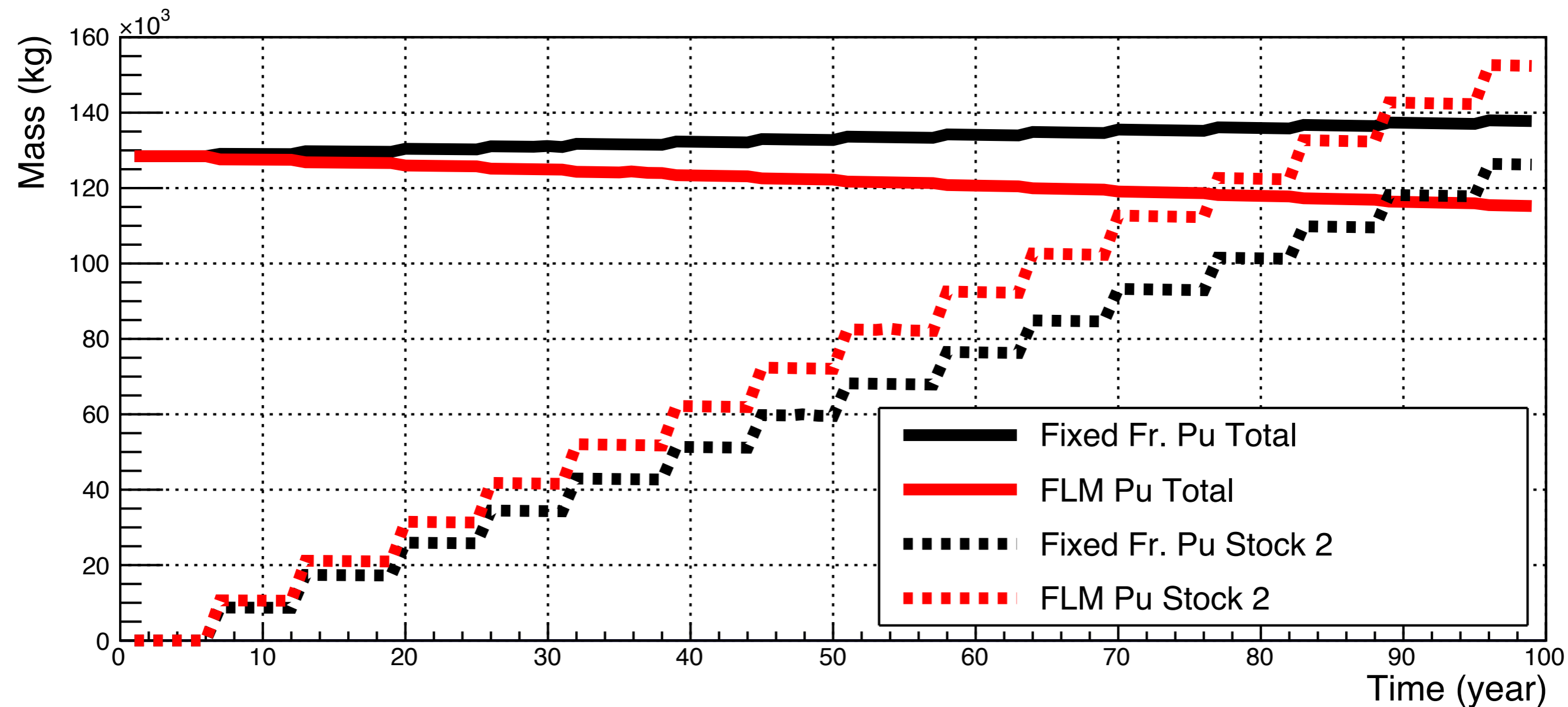
# Results for PWR - MOx Compo 3 / CLASS



- Example of questions strongly impacted by the fuel loading method
  - We suppose we need 100 tons of Pu to start a new technology.
  - When will the deployment be possible?
  - ▶ Fixed Fraction answer : 90 years VS Fuel Loading Model answer : 50 years.



# Results for SFR - MOx Compo 2 / CYCLUS



- The deviation between Total Plutonium FF and FLM is strong
  - ▶ SFR is sur-generator or burner
- The deviation between Stock 2 Plutonium FF and FLM is smaller / PWR
  - ▶ Impact for deployment strategy is smaller

# Conclusions and Perspectives

- FIT Project has started since 2 years as a collaboration France - US
  - 2016 : Madison University and CNRS
  - 2018 : ANL, ORNL and INL integration
  - ▶ Lack of time for results integration
- FIT Project is opened to any fuel cycle simulation tool
  - Increase range of functionalities
  - Small investment, around 2 weeks per year
- 2018 - 2019 : Closing the first functionality tests
  - Run with all fuel cycle simulators involved
  - Formulation of conclusions for range of application of Fuel Loading Model
- Later : Test other functionalities
- Later : Build a dedicated framework for results presentation

# Conclusions and Perspectives

**BACKUP**

# Results from ORNL / ORION

## ● PWR Model used in ORION

- One reference composition for cross sections libraries
- Burnup-dependent cross section from SCALE code
- Westinghouse 17 by 17 LOPAR fuel assembly
- 27 burnup steps

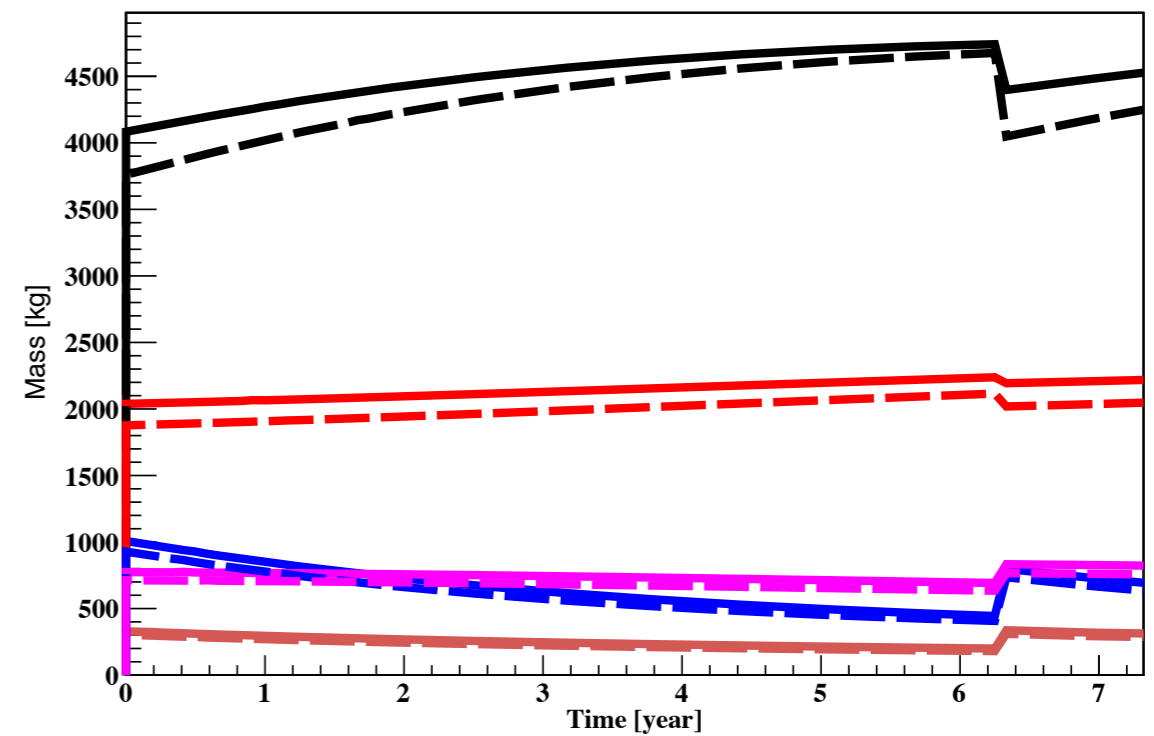
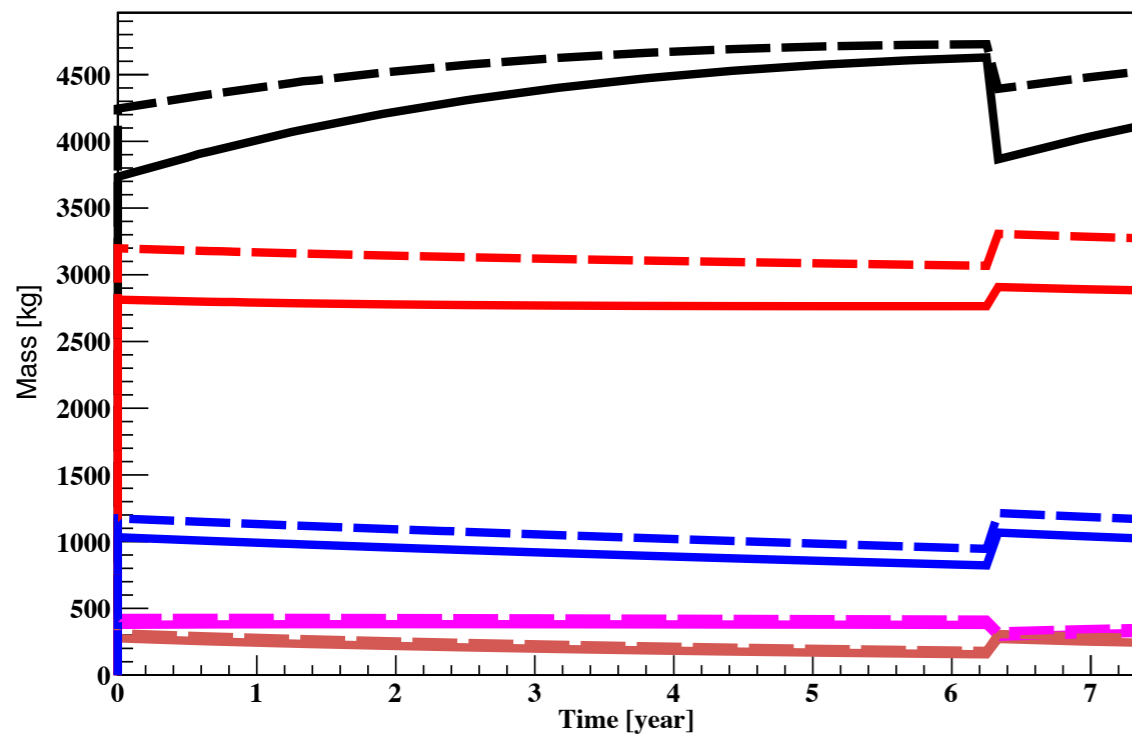
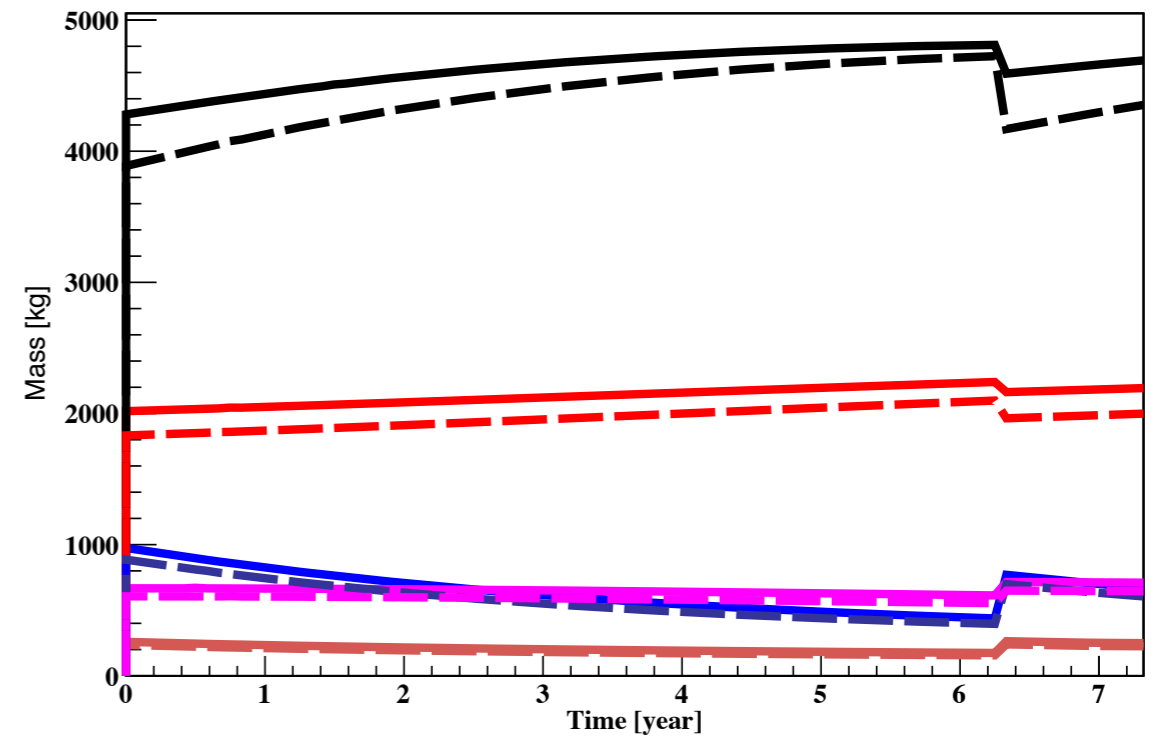
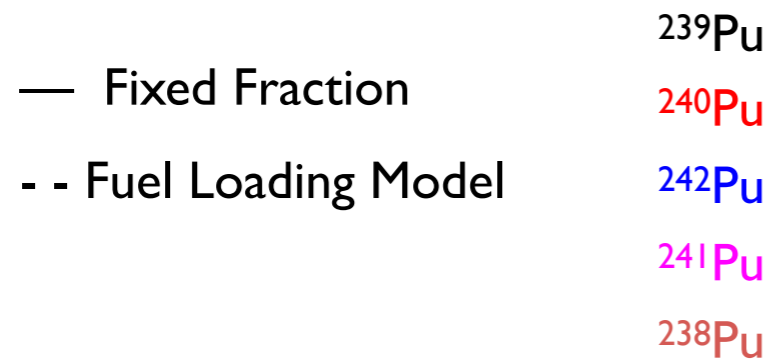
wt%	ref	% diff	1	% diff	2	% diff	3	% diff
<b>U234</b>	0.002	-4.07	0.002	33.1	0.003	-4.35	0.005	-4.89
<b>U235</b>	0.117	0.0650	0.108	5.81	0.099	6.33	0.099	8.98
<b>U236</b>	0.023	-1.03	0.024	-2.50	0.026	-6.29	0.026	-6.32
<b>U238</b>	90.5	-0.400	89.8	0.193	89.8	0.051	89.8	0.076
<b>NP237</b>	0.016	-inf	0.015	-inf	0.019	-inf	0.019	-inf
<b>PU236</b>	0.000	0.000	0.000	0.000	0.000	0.000	0.000	0.000
<b>PU238</b>	0.120	3.99	0.205	0.875	0.170	1.29	0.320	3.19
<b>PU239</b>	2.25	-0.55	2.17	-10.7	1.83	-5.43	1.76	-5.97
<b>PU240</b>	1.62	-6.97	1.45	0.622	1.63	5.60	1.35	-3.31
<b>PU241</b>	0.862	4.37	0.910	1.68	0.953	-1.49	0.863	4.15
<b>PU242</b>	0.436	5.00	0.549	19.6	0.700	12.7	0.814	21.3
<b>AM241</b>	0.059	5.98	0.078	2.16	0.0690	-8.99	0.116	-0.510
<b>Am+Pu</b>	5.48	-3.19	5.59	-4.95	5.61	-2.83	5.52	-3.21

$$\% \text{ diff} = \frac{\text{Cyclus result} - \text{ORION result}}{\text{Cyclus result}} * 100$$

Table 5: Discharge fuel composition for MOX LWR with percent differences from Cyclus results

- Change in plutonium stream quality does affect the ORION output depletion
- Important deviation could be observed between ORION and Cyclus

# FIT case #1 : SFR



# FIT case #2 : PWR

